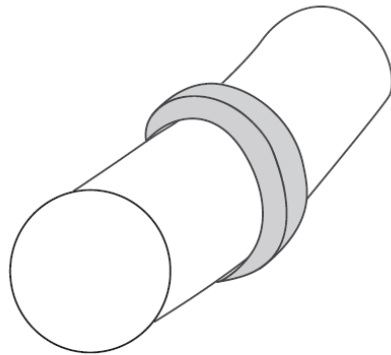


The Spherical tokamak

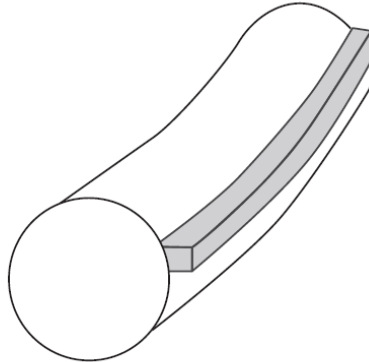


- Aspect ratio $R_0/a \sim 1.6$
- Advantages:
 - Higher β_t limit.
 - A compact design almost spherical in appearance.
- Challenges:
 - Minimum space is given in the center of the torus to accommodate the toroidal field coils.
 - With a very compact design the technology associated with the construction and maintenance of the device may be more difficult than for a “normal” tokamak.
 - Large currents will have to be driven noninductively, a costly and physically difficult requirement.

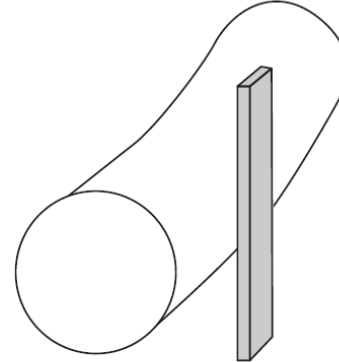
Limiter protects the vacuum chamber from plasma bombardment and defines the edge of the plasma



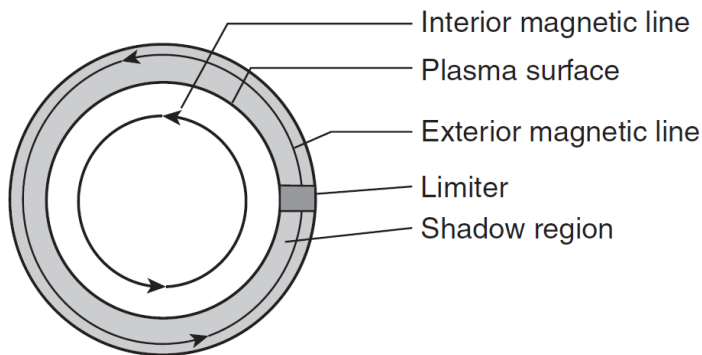
Poloidal limiter



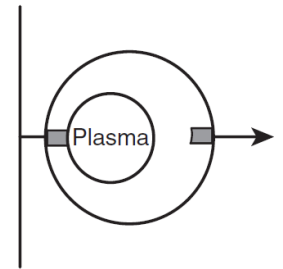
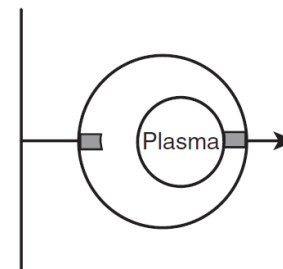
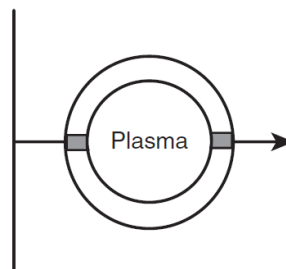
Toroidal limiter



Rail limiter



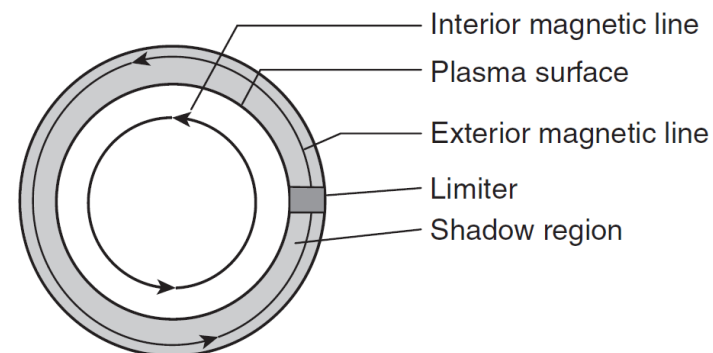
- **Vertical field is correct.**
- **Vertical field is too small.**
- **Vertical field is too large.**



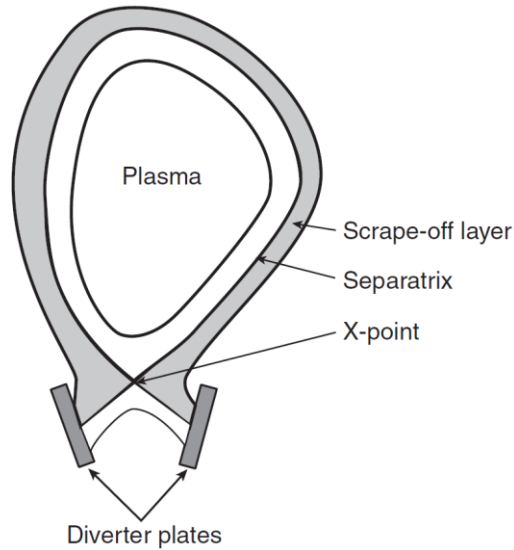
Limiter protects the vacuum chamber from plasma bombardment and defines the edge of the plasma



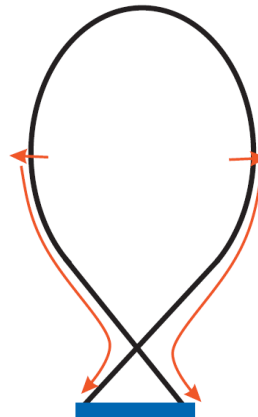
- A mechanical limiter is a robust piece of material, often made of tungsten, molybdenum, or graphite placed inside the vacuum chamber.
- Some of the particles of the limiter surface may escape. Neutral particles can penetrate some distance into the plasma before being ionized.
- The high-z impurities can lead to significant additional energy loss in the plasma through radiation.
- In ignition experiments and fusion reactors, the bombardment is more intense and extends over longer periods of time. In addition, if the impurity level is too high, it may not be possible to achieve a high enough temperature to ignite.



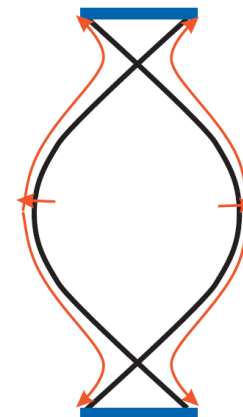
The magnetic divertor – guide a narrower layer of magnetic lines away from the edge of the plasma



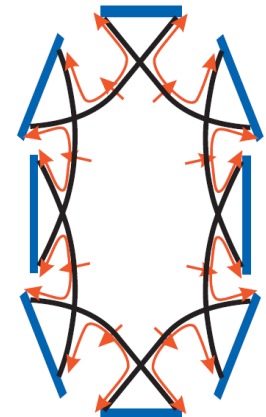
- **Single-null poloidal-field divertors for tokamak**



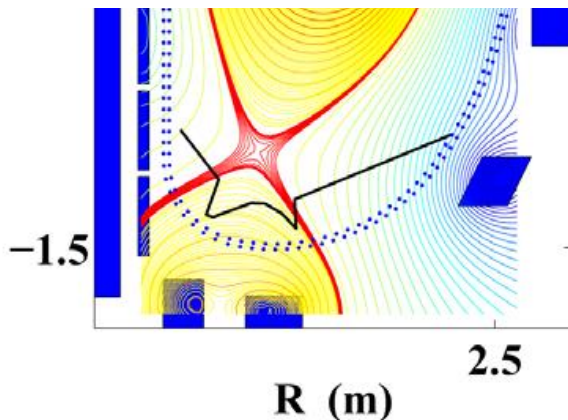
- **Double-null poloidal-field divertors for tokamak**



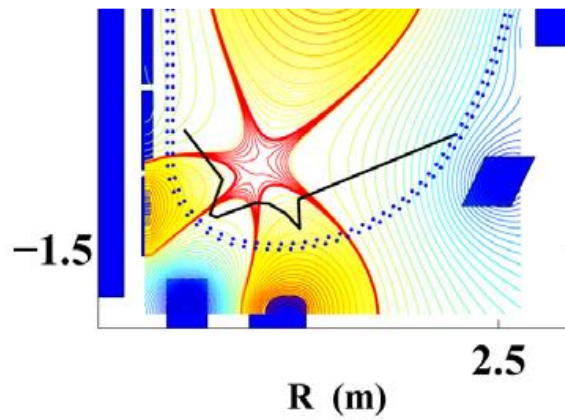
- **Island divertor for stellerators**



- **Standard**



- **Snowflake**



Y. Feng, Nucl. Fusion, **46**, 807 (2006)
 L Xue *et al*, Plasma Phys. Control. Fusion **58**, 055005 (2016)

Pros and cons of a divertor



- **Advantages:**
 - The collector plate is remote from the plasma. There is space available to spread out the magnetic lines.
 - A lower intensity of particles and energy bombard the collector plate leading to a longer replacement time.
 - It is more difficult for impurities to migrate into the plasma.
 - There are longer distance distances to travel and if a neutral particle becomes ionized before or during the time it crosses the divertor layer on its way toward the plasma, its parallel motion then carries it back to the collector plate.
 - The larger divertor chamber provides more access to pump out impurities.
 - The plasma edge is not in direct contact with a solid material such as a limiter.
- **Disadvantages:** larger and more complex system and more expensive.

Course Outline

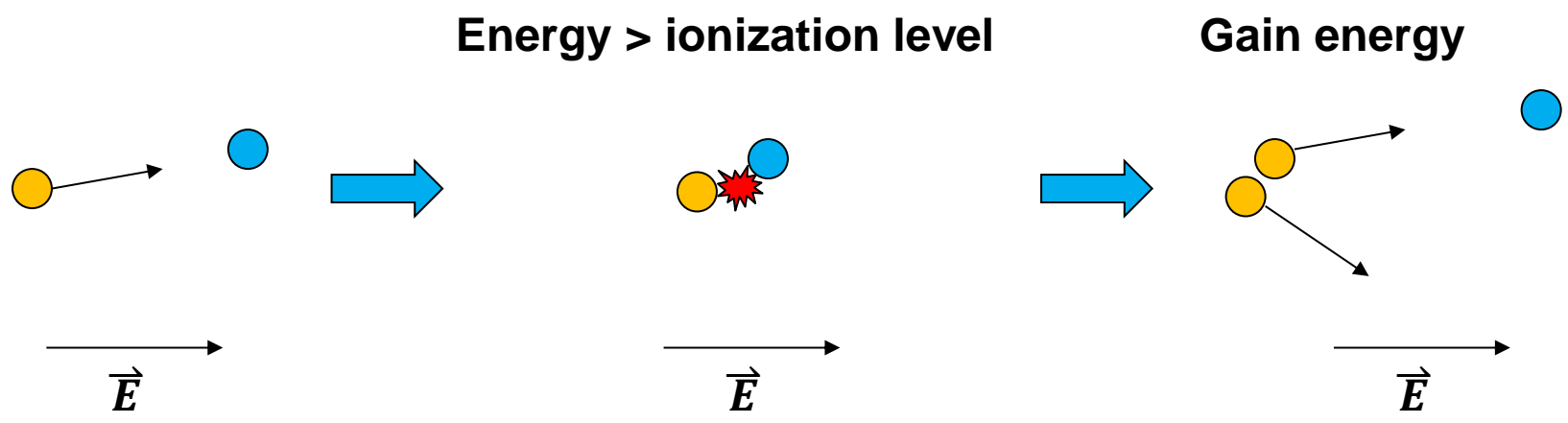


- **Magnetic confinement fusion (MCF)**
 - Gyro motion, MHD
 - 1D equilibrium (z pinch, theta pinch)
 - Drift: ExB drift, grad B drift, and curvature B drift
 - Tokamak, Stellarator (toroidal field, poloidal field)
 - Magnetic flux surface
 - 2D axisymmetric equilibrium of a torus plasma: Grad-Shafranov equation.
 - Stability (Kink instability, sausage instability, Safety factor Q)
 - Central-solenoid (CS) start-up (discharge) and current drive
 - CS-free current drive: electron cyclotron current drive, bootstrap current.
 - Auxiliary Heating: ECRH, Ohmic heating, Neutral beam injection.

Collisions play an important role in ionization process



- At the microscopic level, breakdown requires the presence of sufficiently energy charge particles that have acquired enough energy from the applied electric field between two energy-dissipating collisions to ionize the material and to create more charge particles.



In most cases, electrons dominate the breakdown process since its mobility is much larger than that of ions



$$E_k = \frac{1}{2}mv^2$$

$$v = \sqrt{\frac{2E_k}{m}}$$

$$E_k \sim kT$$

Collision time: $t = \frac{s}{\sqrt{\frac{2E_k}{m}}} \sim \frac{n^{-1/3}}{\sqrt{T}} \sqrt{m}$

$$n = \frac{\#}{V} \sim \frac{\#//}{s^3}$$

$$s \sim n^{-1/3}$$

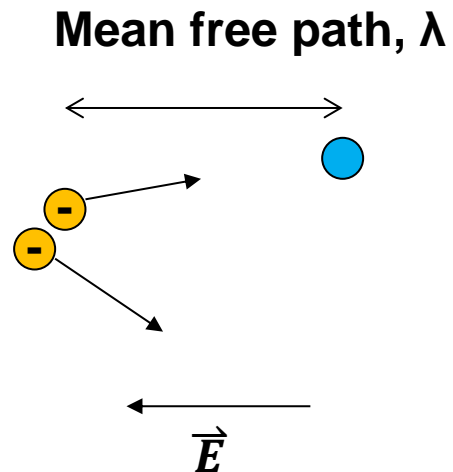
$$\frac{m_i}{m_e} \sim 2000 \times \text{Atomic mass}$$

$$\frac{t_i}{t_e} \sim 45 \times \sqrt{A}$$

Mean free path is important in ionization process



- For an electron to acquire enough energy between collisions, its mean free path in the material must be sufficiently long.

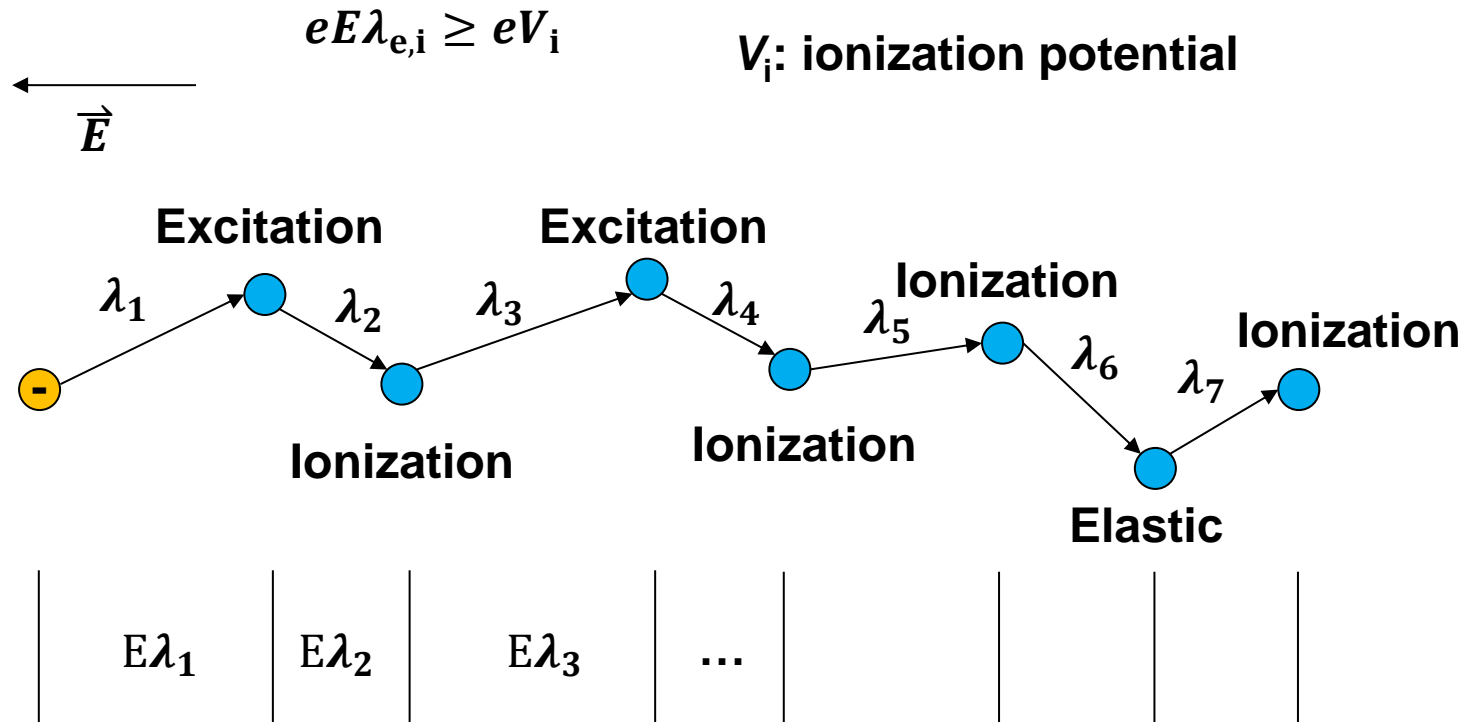


$$E_k = e \times E \times \lambda = eV$$

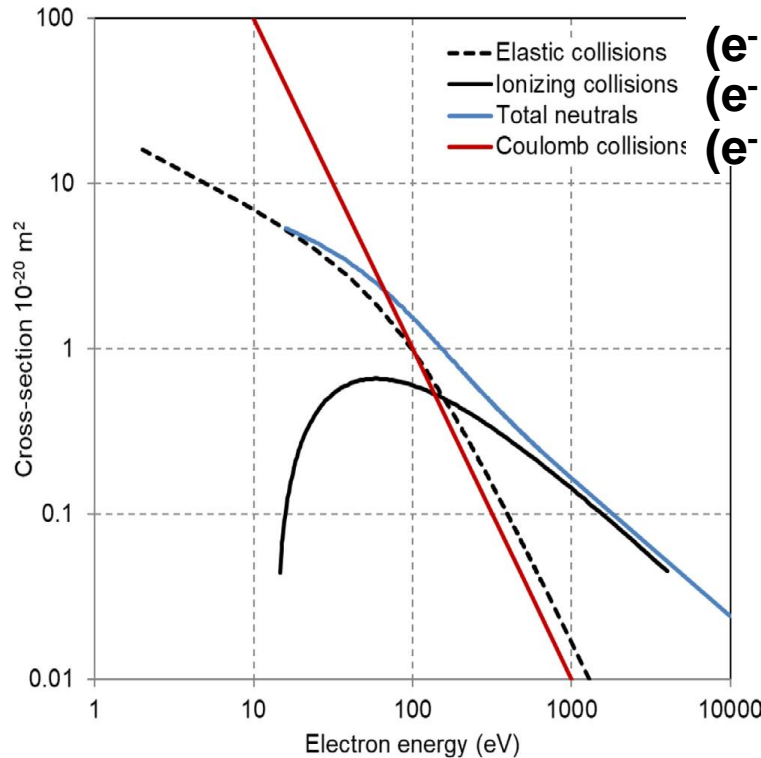
Electron impact ionization is the most important process in a breakdown of gases



- Electron impact ionization: $A + e^- \rightarrow A^+ + e^- + e^-$
 - The most important process in the breakdown of gases but is not sufficient alone to result in the breakdown.



Collision cross-sections of elastic, ionizing collisions between e^- and H_2 and coulomb collisions



(e^- vs H_2)
 $(e^- + H_2 \rightarrow 2e^- + H^+ + H)$
 (e^- vs H^+)

$$\sigma_{\text{elastic}, m^2} = \frac{1.75 \times 10^{-16}}{(W_{e, \text{eV}}^{1.5} + 750) \sqrt{W_{e, \text{eV}}}}$$

$$\sigma_{\text{ionizing}, m^2} \sim 3 \times 10^{-20} \left(\ln \epsilon - 0.69 + \frac{0.66}{\epsilon} \right) \epsilon^{-1}$$

$$\epsilon = \frac{W_e}{E_{\text{ion}}} \quad E_{\text{ion}} \sim 15 \text{ eV for } H_2$$

$$\sigma_{\text{coulomb}} = \frac{e^4}{4\pi\epsilon_0^2 m_e^2 v_e^2} \ln \Lambda$$

$$= \frac{e^4}{16\pi\epsilon_0^2 W_e^2} \ln \Lambda \sim 10^{-16} W_e^{-2}$$

for $\ln \Lambda \sim 13 - 15$

$$\nu = n v_E \sigma$$

$$\lambda = \frac{1}{n \sigma}$$

Townsend avalanche process for Tokamak breakdown



- The first Townsend coefficient α : the number of ionizing collisions made on the average by an electron as it travels 1 m along the electric field:

$$\alpha \sim \frac{1}{\lambda_i} = \frac{\nu_{ei}}{\bar{v}_e} = \frac{n_0 \langle \sigma v_e \rangle_{ne}}{\bar{v}_e} = \frac{p}{T} \frac{\langle \sigma v \rangle_{ne}}{\bar{v}_e} \equiv Ap \quad A \equiv \frac{1}{T} \frac{\langle \sigma v \rangle_{ne}}{\bar{v}_e}$$

- Number of primary electrons with energy higher than the ionization potential:

$$dn_e = -n_e \frac{dx_i}{\lambda_i} \Rightarrow \frac{n_e(x_i)}{n_{e0}} = \exp\left(-\frac{x_i}{\lambda_i}\right)$$

$$\alpha \equiv \frac{\text{\# / ionization collisions}}{\text{per electron}} \times (\text{\# / electron with } E > \text{ionization potential})$$

$$= \frac{1}{\lambda_i} \frac{n_e(x_i)}{n_{e0}} = \frac{1}{\lambda_i} \exp\left(-\frac{x_i}{\lambda_i}\right)$$

$$\alpha = Ap \exp(-Ap x_i)$$

$$A = 3.83 \text{ m}^{-1} \text{Pa}^{-1} = 1060 \text{ m}^{-1} \text{Torr}^{-1}$$

$$B = 96.6 \text{ Vm}^{-1} \text{Pa}^{-1} = 35000 \text{ m}^{-1} \text{Torr}^{-1}$$

$$\alpha = Ap \exp\left(-\frac{AV^*}{E/p}\right) \equiv Ap \exp\left(-\frac{B}{E/p}\right) \quad x_i \approx \frac{V^*}{E} \text{ where } V^* > V_i$$

- The parameters A and B must be experimentally determined.

Paschen-like curve for minimum breakdown voltage



$$\alpha \sim \frac{1}{\lambda_i} = Ap \quad \alpha = Ap \exp\left(-\frac{B}{E/p}\right)$$

$$A = 3.83 \text{ m}^{-1}\text{Pa}^{-1} = 1060 \text{ m}^{-1}\text{Torr}^{-1}$$

$$B = 96.6 \text{ Vm}^{-1}\text{Pa}^{-1} = 35000 \text{ m}^{-1}\text{Torr}^{-1}$$

- For $p=1 \text{ mPa}$, $\lambda_i \sim 262 \text{ m}$, for ITER, $2\pi r_o \sim 38 \text{ m}$ $\frac{\lambda_i}{2\pi r_o} \sim 7$

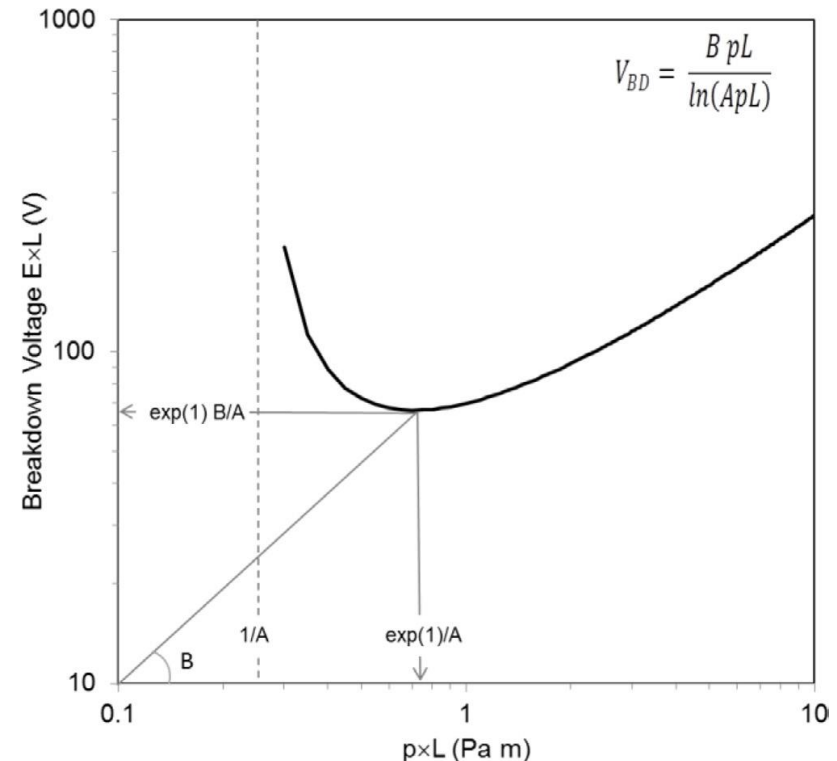
- For breakdown to happen:

$$\alpha L > 1 \quad \alpha L = ApL \exp\left(-\frac{BpL}{V_{BD}}\right) > 1$$

$$\exp\left(-\frac{BpL}{V_{BD}}\right) > \frac{1}{ApL}$$

$$-\frac{BpL}{V_{BD}} > -\ln(ApL)$$

$$V_{BD} > \frac{BpL}{\ln(ApL)} \quad E_{BD} > \frac{Bp}{\ln(ApL)}$$



Perpendicular stray-field (B_z) needs to be as small as possible



- For $p=1$ mPa, $\lambda_i \sim 262$ m, for ITER, $2\pi r_o \sim 38$ m $\frac{\lambda_i}{2\pi r_o} \sim 7$

$$\frac{B_z}{B_T} \sim 10^{-3} \quad \lambda_i \times \frac{B_z}{B_T} = 0.26 \text{ m}$$

- For ITER,

$$E \sim E_{\text{loop}} = 0.3 \text{ V/m}$$

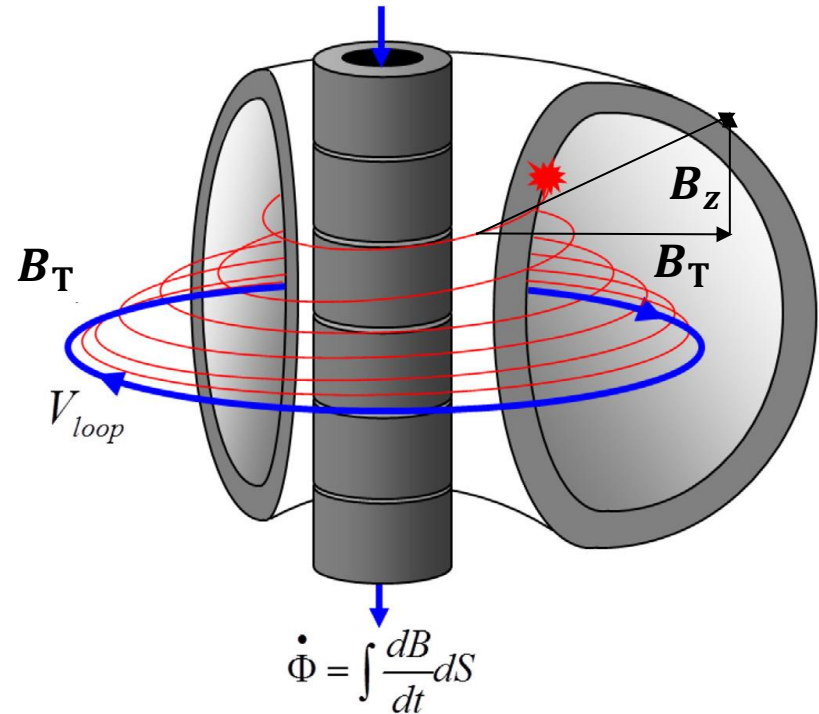
$$p = 1 \text{ mPa} \quad L_{\text{BD}} = 357 \text{ m}$$

- Required loop field:

$$E_{\text{BD}} > \frac{Bp}{\ln(ApL)}$$

$$E_{\text{BD}} > \frac{1.25 \times 10^4 P_{\text{Torr}}}{\ln(510PL_c)}$$

$$L_c = 0.25 a_{\text{eff}} \left(\frac{B_z}{B_T} \right)$$



- W/ preionization: $E_T \frac{B_T}{B_z} \geq 100 \text{ V/m}$
- Purely Ohmic discharges: $E_T \frac{B_T}{B_z} \geq 1000 \text{ V/m}$

Examples or required loop electric fields

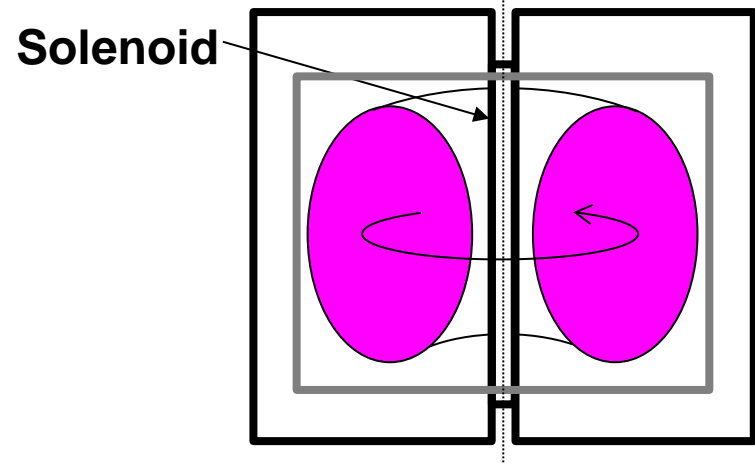
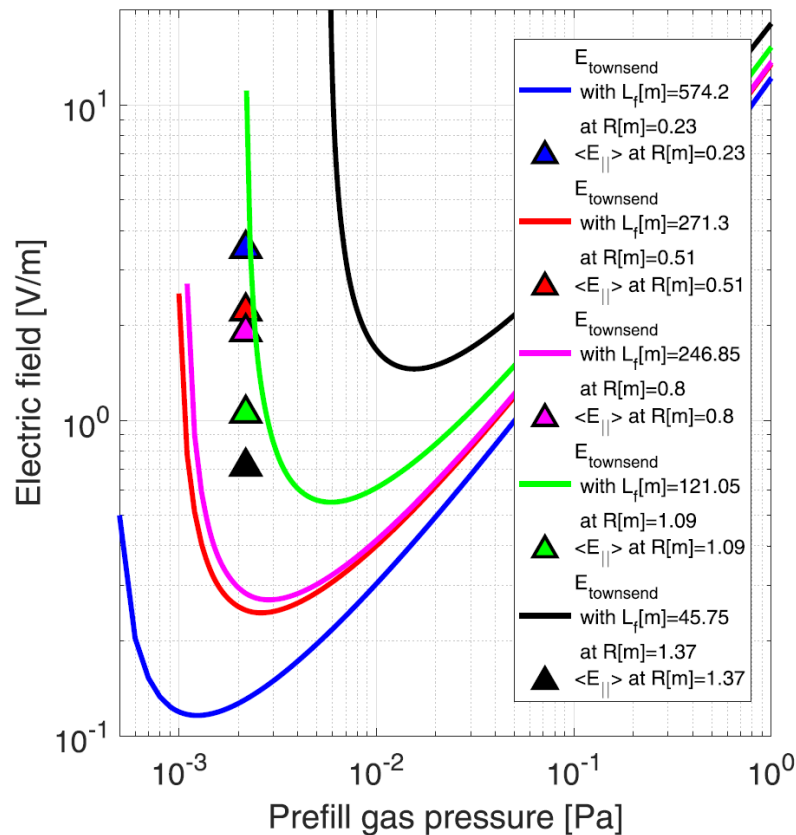


$$E_{BD} > \frac{1.25 \times 10^4 P_{Torr}}{\ln(510 P L_c)}$$

$$L_c = 0.25 a_{eff} \left(\frac{B_z}{B_T} \right)$$

- W/ preionization: $E_T \frac{B_T}{B_z} \geq 100 \text{ V/m}$

- Purely Ohmic discharges: $E_T \frac{B_T}{B_z} \geq 1000 \text{ V/m}$



$$V_\phi = -\frac{\partial \phi}{\partial t} \equiv M \frac{\partial I_{CS}}{\partial t}$$

$$E_\phi = -\frac{1}{2\pi r} \frac{\partial \phi}{\partial t}$$

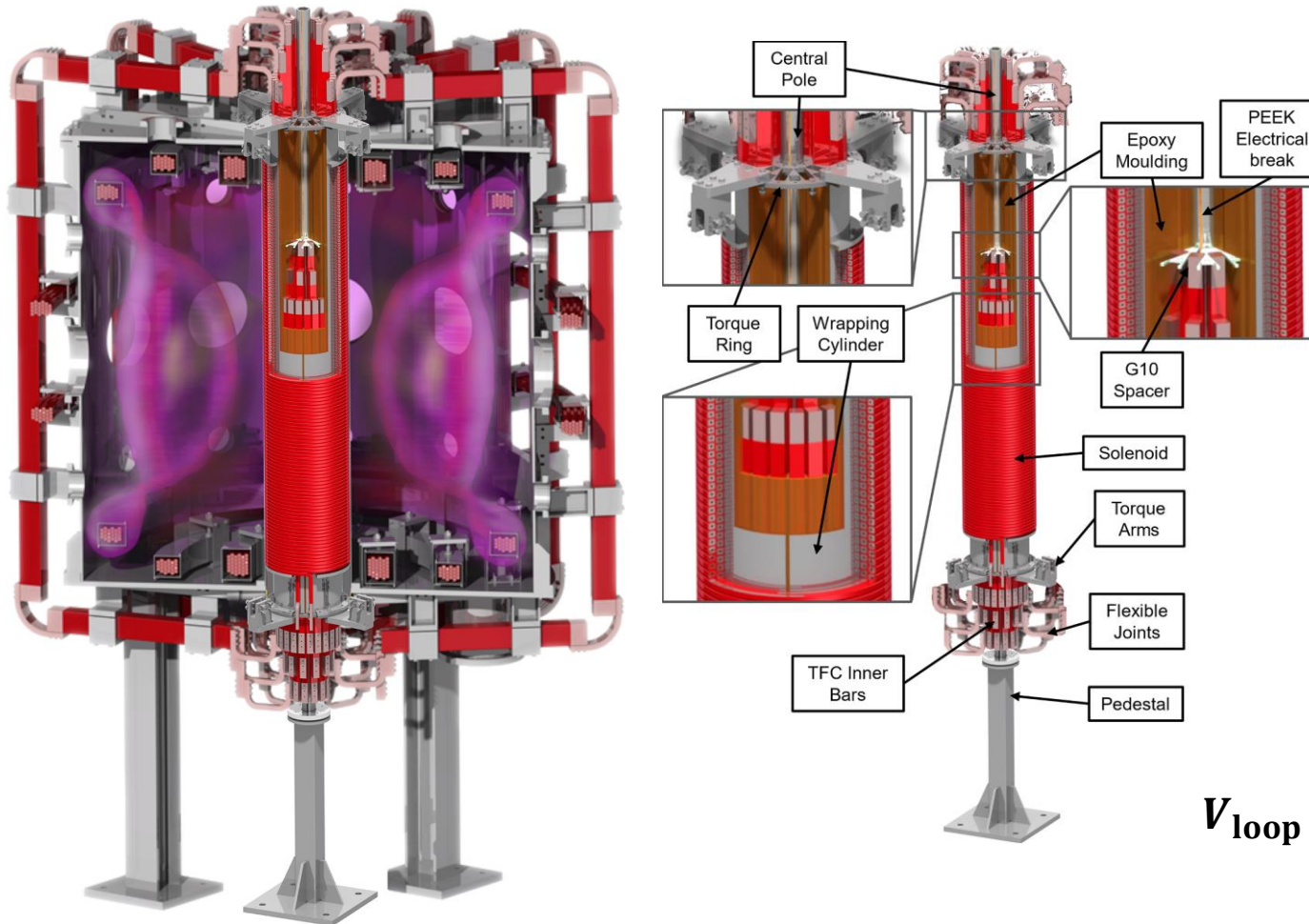
H.-T. Kim, etc., Nucl. Fusion **62**, 126012 (2022)

S. J. Doyle et al, Fusion Eng. Des. **171**, 112706 (2021)

Central solenoid can be used to provide the required loop voltage for breakdown

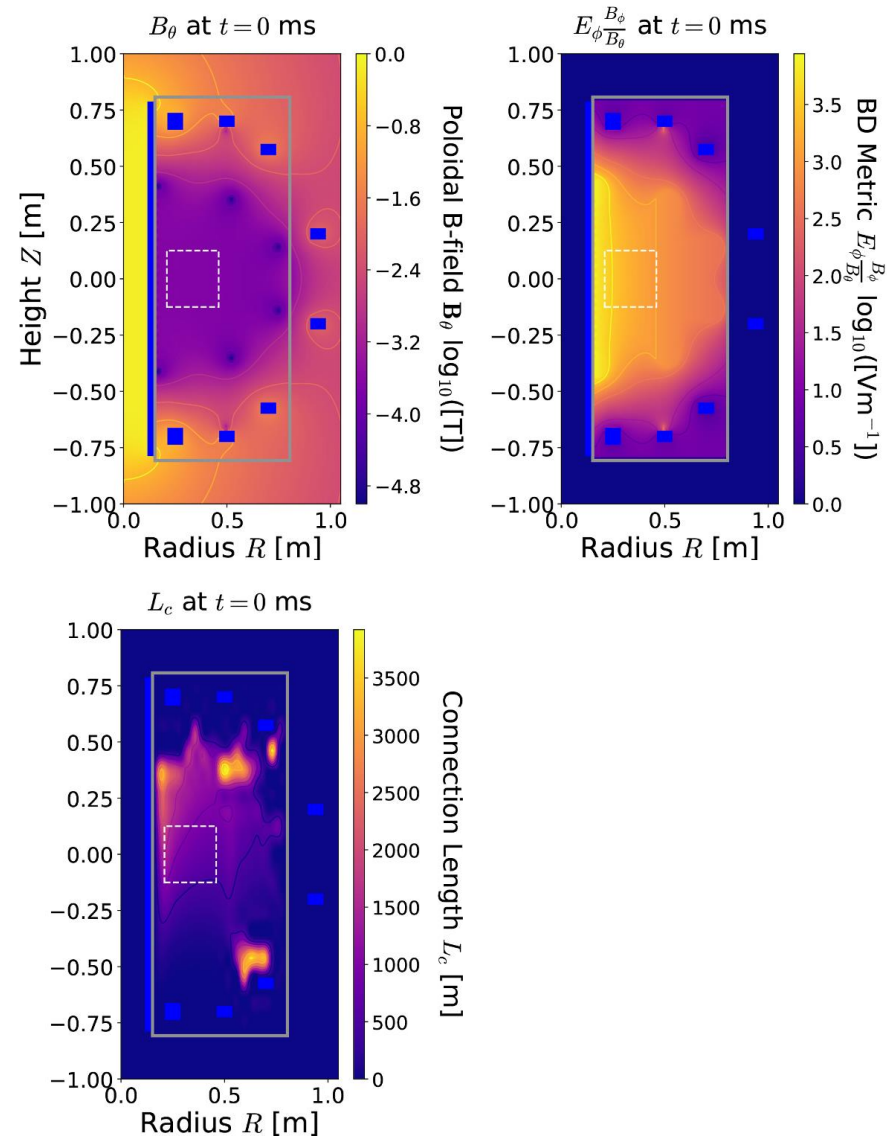


- SMART:

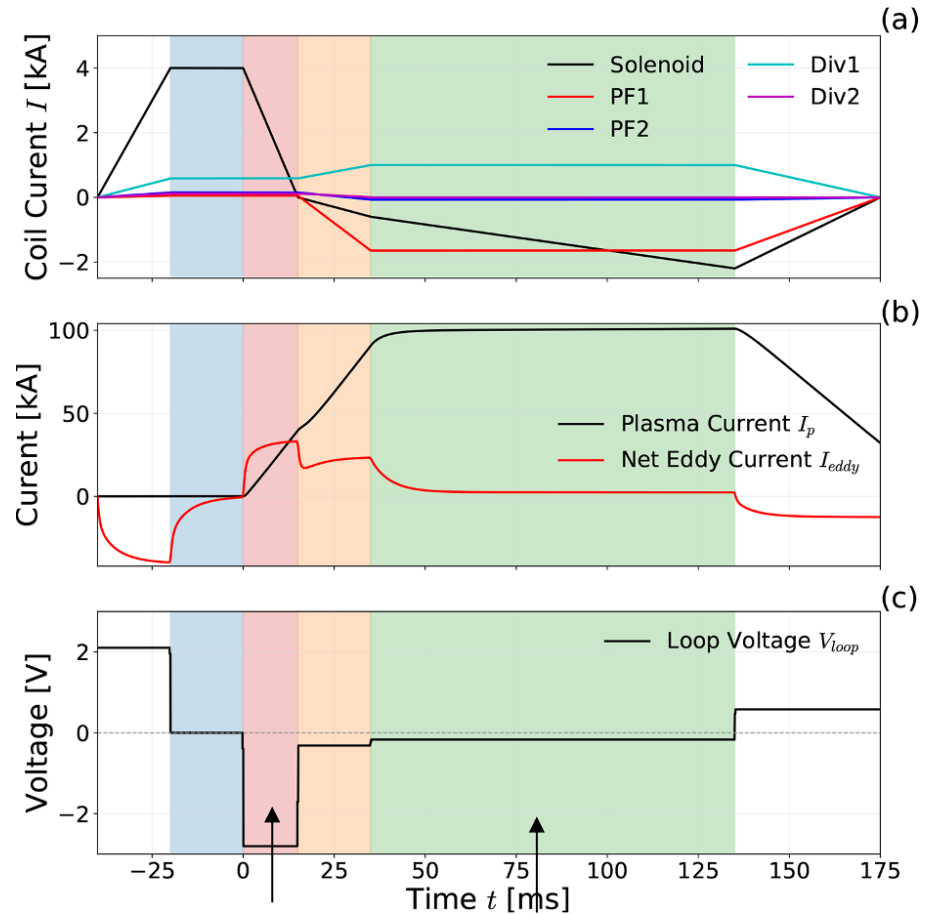


$$V_{\text{loop}} = \frac{A_{\text{sol}} \mu N_{\text{sol}}}{L_{\text{sol}}} \frac{dI_{\text{sol}}}{dt}$$

Poloidal coils are used to reduce the stray field during breakdown

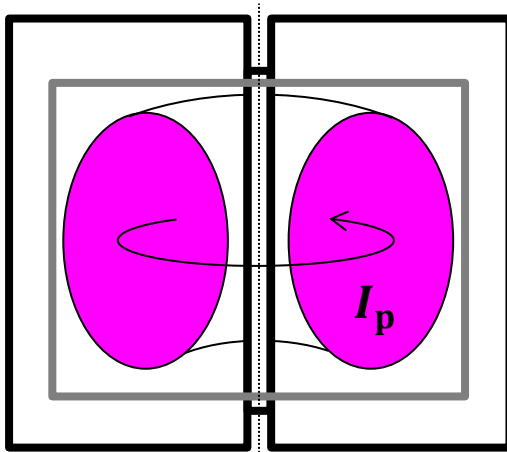


• Currents of SMART

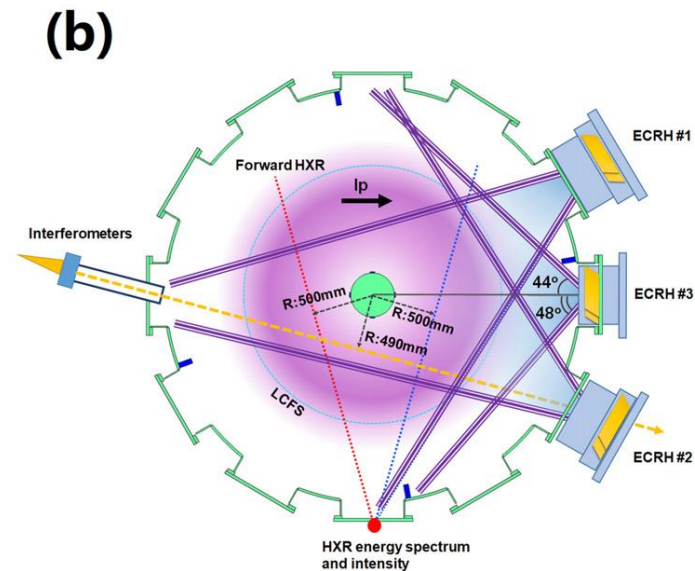
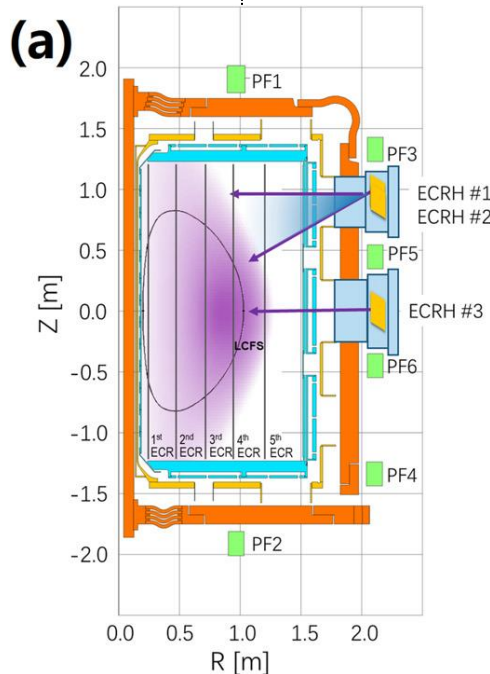


Breakdown **Plasma current is driven**

Momentum exchange may be needed to drive plasma current



$$\vec{j}_p = \Sigma q n \vec{v} = -en_e \vec{v}_e + en_i \vec{v}_i$$



Solenoid can be used to drive the plasma current

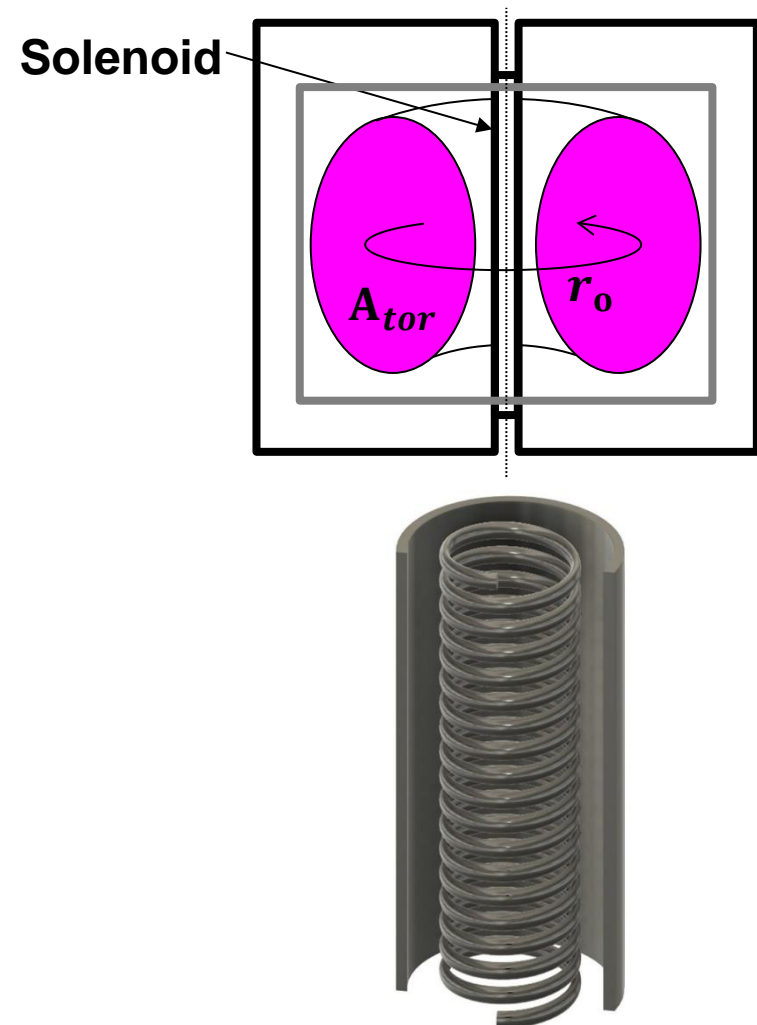


$$L_{\text{tor}} \frac{dI}{dt} + IR = V_{\text{loop}} = M \frac{dI_{\text{sol}}}{dt}$$

$$L_{\text{tor}} = \mu_0 r_0 \left[\ln \left(\frac{8r_0}{a} \right) - 1.5 \right]$$

$$R_{\text{spitzer}} = \eta_{\text{spitzer}} \frac{2\pi r_0}{A_{\text{tor}}}$$

$$\eta_{\text{spitzer}} = 5.2 \times 10^{-3} Z \ln \Lambda T_{e,(\text{eV})}^{-3/2}$$



Current is initially driven at the surface and then diffuses into the plasma



- Simplified Ohm's law: $\vec{E} + \vec{v} \times \vec{B} = \eta \vec{j}$
- Assuming a stationary plasma: $\nabla \times \vec{E} = -\frac{\partial \vec{B}}{\partial t} = \nabla \times (\eta \vec{j})$
- Assuming a constant η :

$$-\frac{\partial}{\partial t} \nabla \times \vec{B} = \eta \nabla \times \nabla \times \vec{j} = \eta (\nabla (\nabla \cdot \vec{j}) - \nabla^2 \vec{j})$$

$$\frac{\partial \vec{j}}{\partial t} = \frac{\eta}{\mu_0} \nabla^2 \vec{j}$$

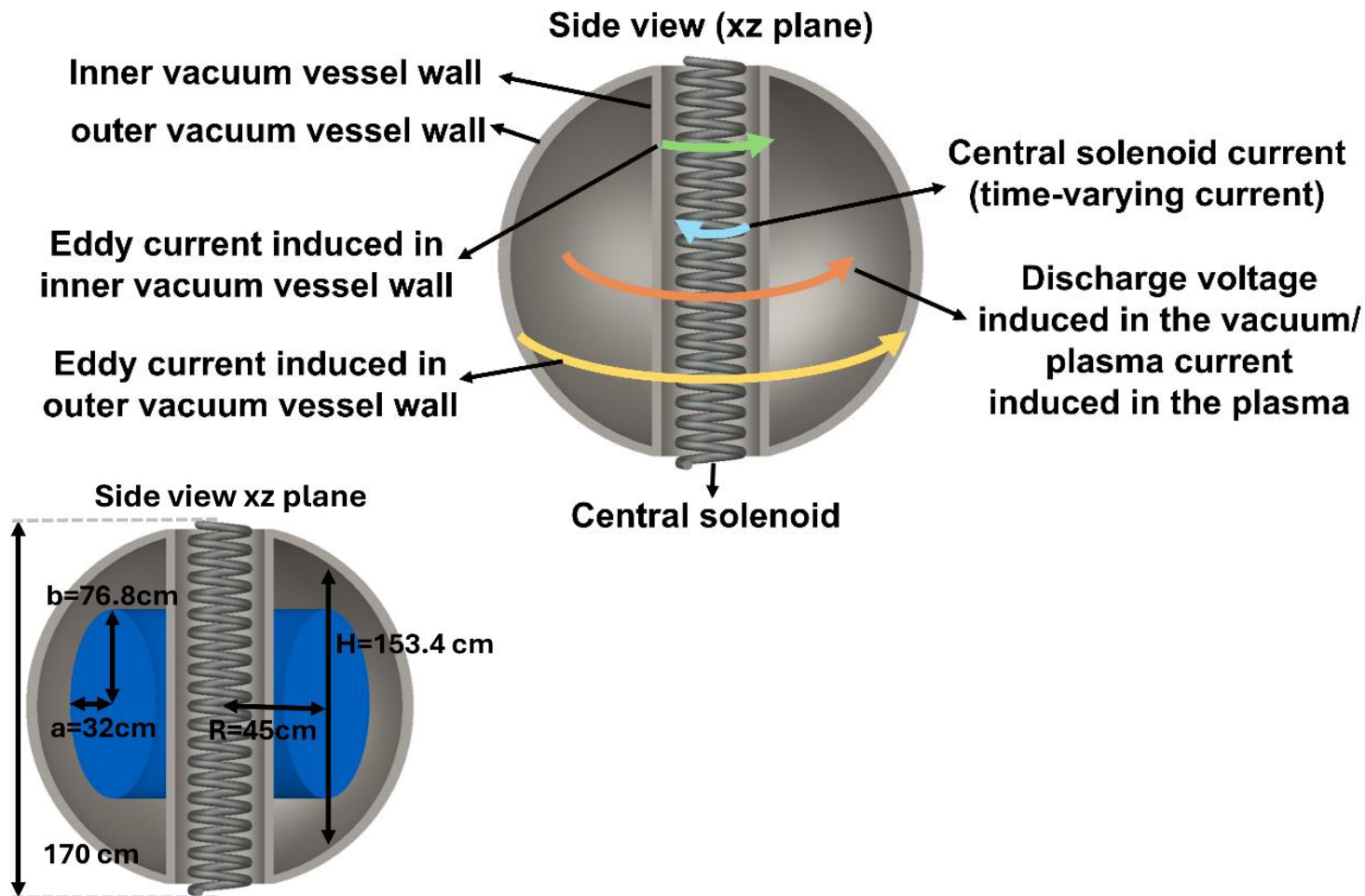
- Assuming non-constant η :

$$\frac{\partial \vec{j}}{\partial t} = \frac{1}{\mu_0} \nabla^2 (\eta \vec{j}) - \nabla [\nabla \cdot (\mu \vec{j})]$$

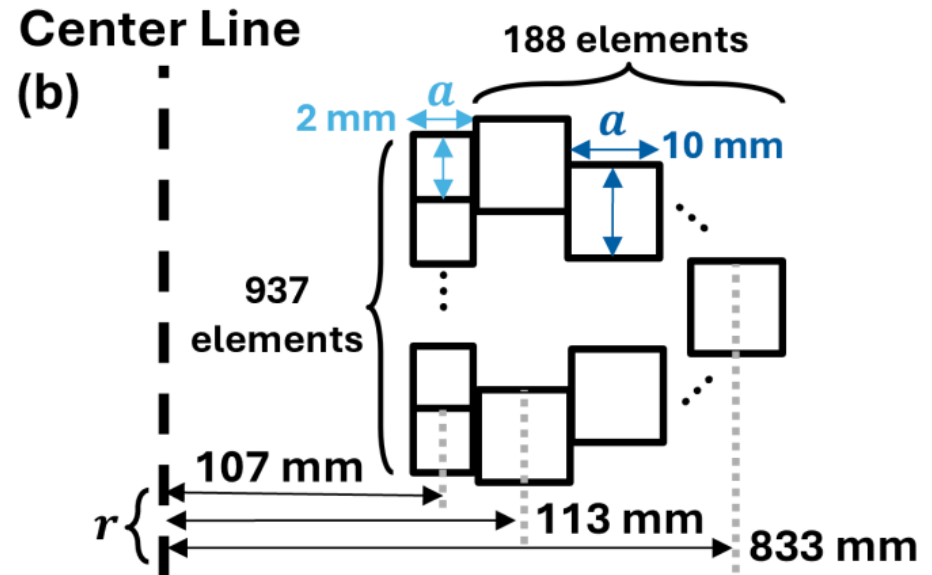
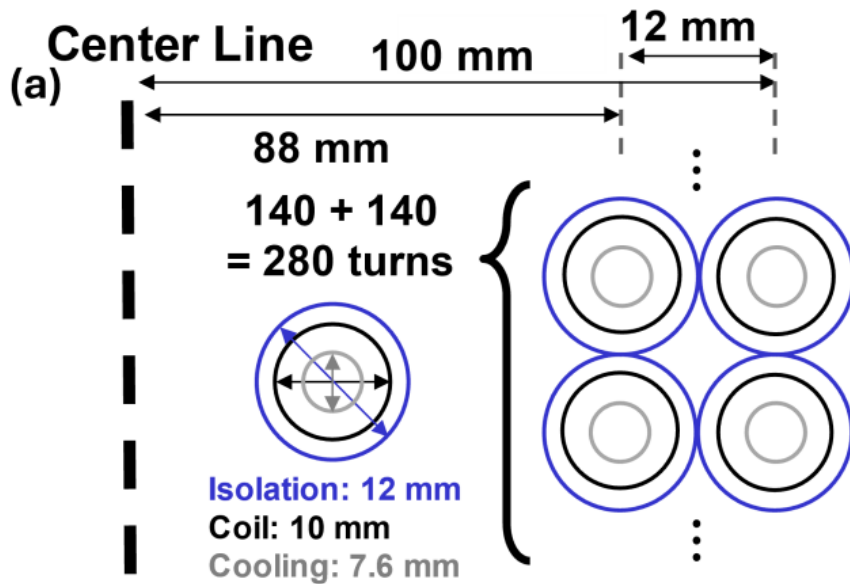
$$\frac{\partial j_T}{\partial t} = \frac{1}{\mu_0} \nabla^2 (\eta j_T)$$

- Since $\eta \propto T^{-3/2}$, resistance drops with higher temperature. The typical limited temperature is ~ 3 keV.

Eddy current needed to be considered



Chamber is broken into ring elements carrying eddy currents



The eddy current in each chamber element is solved numerically

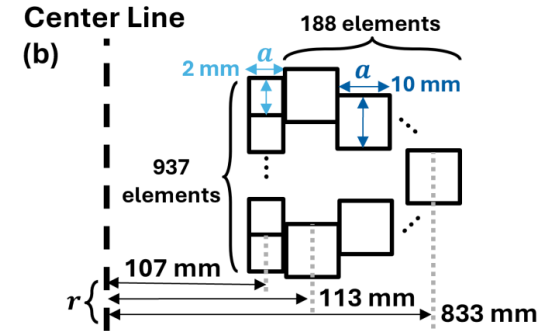


$$\vec{M} \frac{d\vec{I}}{dt} + \vec{R}_\Omega \vec{I} = \vec{V} \quad \vec{M} \frac{\vec{I}' - \vec{I}}{\Delta t} + \vec{R}_\Omega \vec{I} = \vec{V}$$

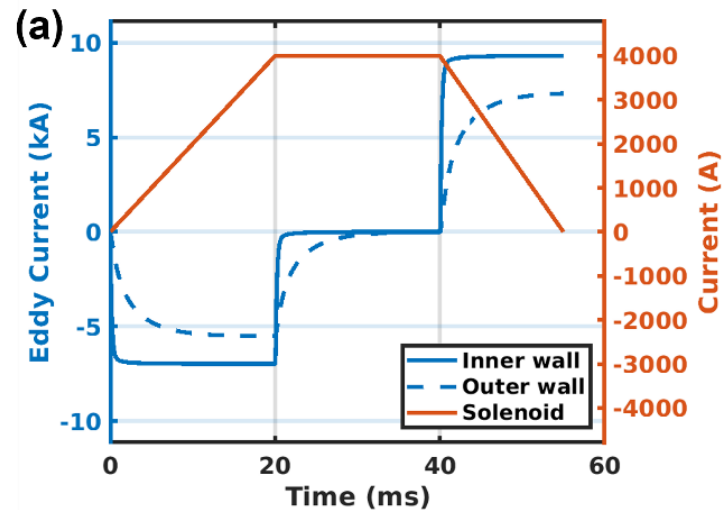
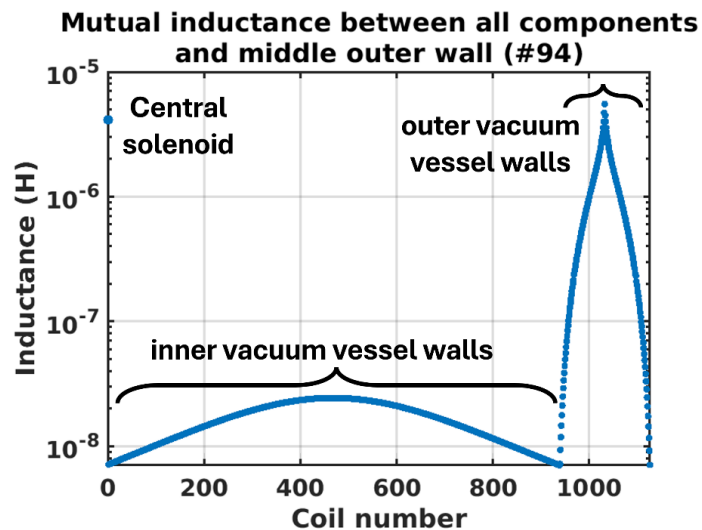
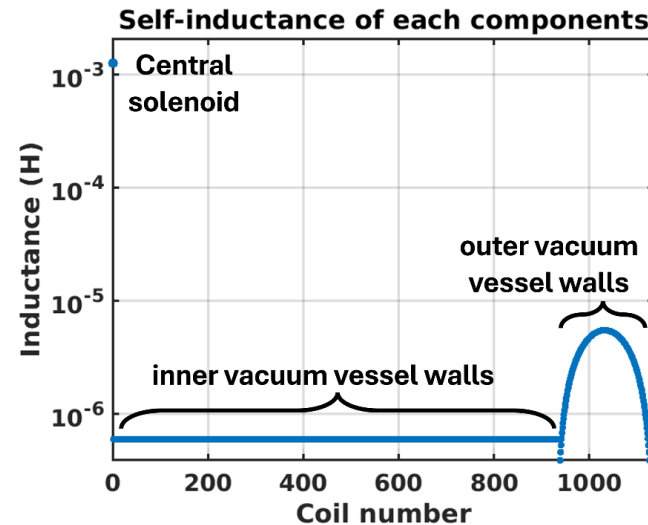
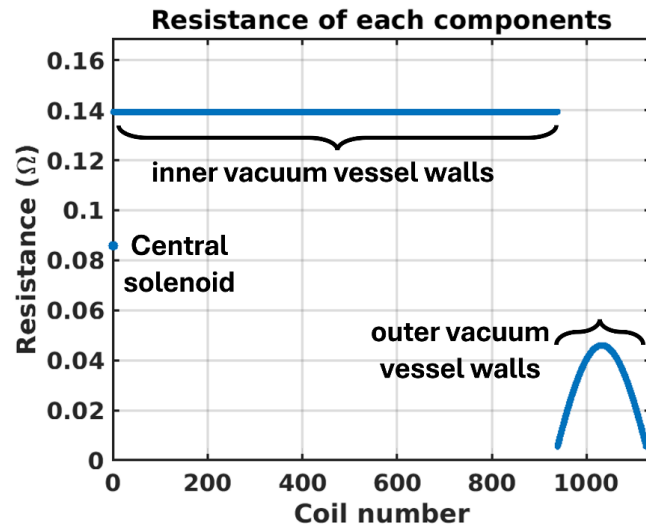
$$\vec{I}' = \left(\vec{1} - \Delta t \vec{M}^{-1} \vec{R}_\Omega \right) \vec{I} + \Delta t \vec{M}^{-1} \vec{V}$$

$$\vec{V} = \vec{M} \frac{d\vec{I}}{dt} + \vec{R}\vec{I}$$

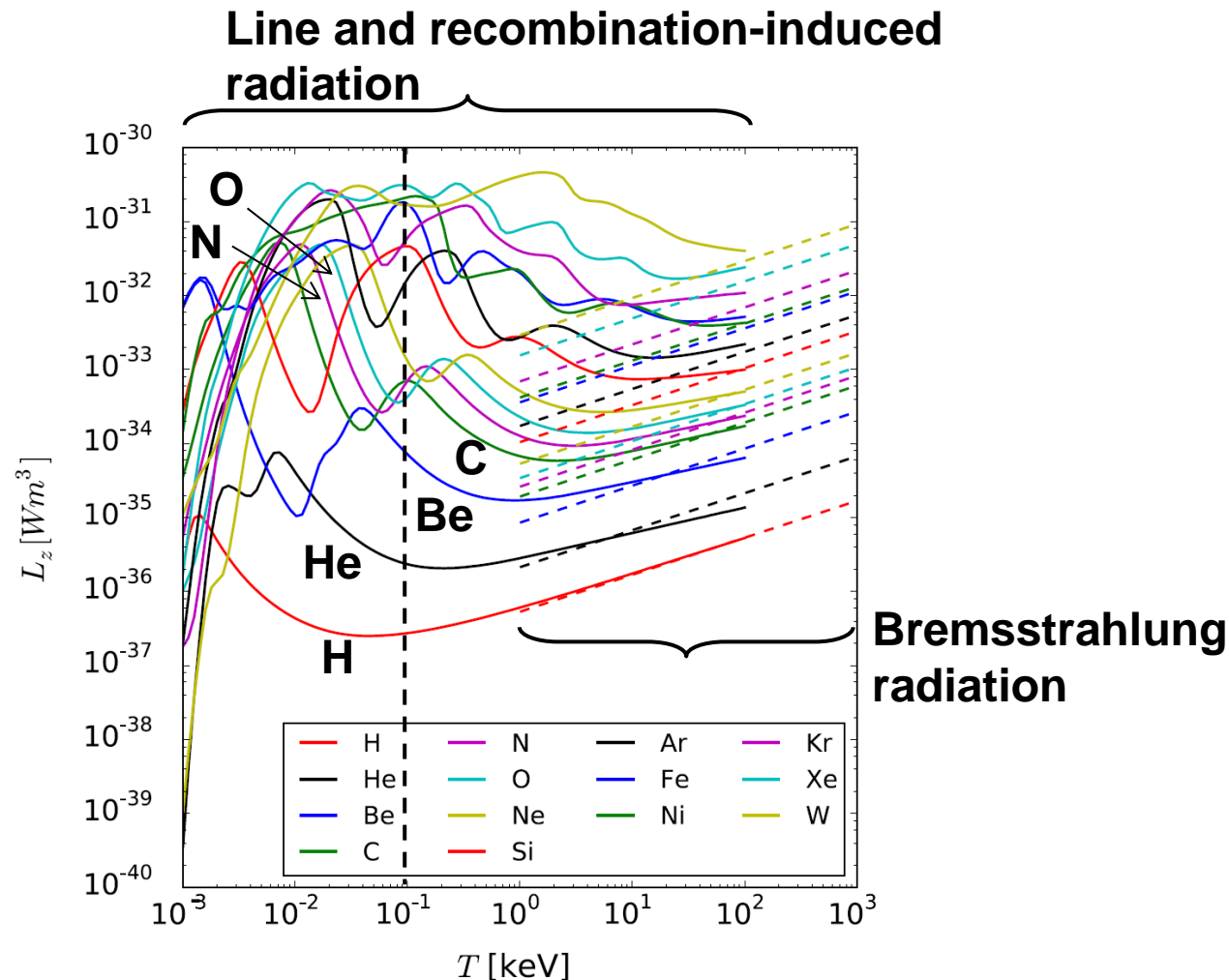
$$\Rightarrow \begin{bmatrix} V_{c1} \\ V_{c2} \\ \vdots \\ V_{v1} \\ V_{v2} \\ \vdots \\ V_p \end{bmatrix} = \begin{bmatrix} L_{c1} & M_{c1,c2} & M_{c1,c3} & \cdots & M_{c1,v1} & M_{c1,v2} & M_{c1,v3} & \cdots & M_{c1,p} \\ M_{c2,c1} & L_{c2} & M_{c2,c3} & \cdots & M_{c2,v1} & M_{c2,v2} & M_{c2,v3} & \cdots & M_{c2,p} \\ M_{c3,c1} & M_{c3,c2} & L_{c3} & \cdots & M_{c3,v1} & M_{c3,v2} & M_{c3,v3} & \cdots & M_{c3,p} \\ \vdots & \vdots & \vdots & \vdots & \vdots & \vdots & \vdots & \vdots & \vdots \\ M_{v1,c1} & M_{v1,c2} & M_{v1,c3} & \cdots & L_{v1} & M_{v1,v2} & M_{v1,v3} & \cdots & M_{v1,p} \\ M_{v2,c1} & M_{v2,c2} & M_{v2,c3} & \cdots & M_{v2,v1} & L_{v2} & M_{v2,v3} & \cdots & M_{v2,p} \\ M_{v3,c1} & M_{v3,c2} & M_{v3,c3} & \cdots & M_{v3,v1} & M_{v3,v2} & L_{v3} & \cdots & M_{v3,p} \\ \vdots & \vdots & \vdots & \vdots & \vdots & \vdots & \vdots & \vdots & \vdots \\ M_{p,c1} & M_{p,c2} & M_{p,c3} & \cdots & M_{p,v1} & M_{p,v2} & M_{p,v3} & \cdots & L_p \end{bmatrix} \begin{bmatrix} I'_{c1} \\ I'_{c2} \\ \vdots \\ I'_{v1} \\ I'_{v2} \\ \vdots \\ I'_p \end{bmatrix} + \begin{bmatrix} R_{c1} & 0 & \cdots & 0 & 0 & \cdots & 0 \\ 0 & R_{c2} & \cdots & 0 & 0 & \cdots & 0 \\ \vdots & \vdots & \vdots & \vdots & \vdots & \vdots & \vdots \\ 0 & 0 & \cdots & R_{v1} & 0 & \cdots & 0 \\ 0 & 0 & \cdots & 0 & R_{v2} & \cdots & 0 \\ \vdots & \vdots & \vdots & \vdots & \vdots & \vdots & \vdots \\ 0 & 0 & \cdots & 0 & 0 & \cdots & R_p \end{bmatrix} \begin{bmatrix} I_{c1} \\ I_{c2} \\ \vdots \\ I_{v1} \\ I_{v2} \\ \vdots \\ I_p \end{bmatrix} \quad (2)$$



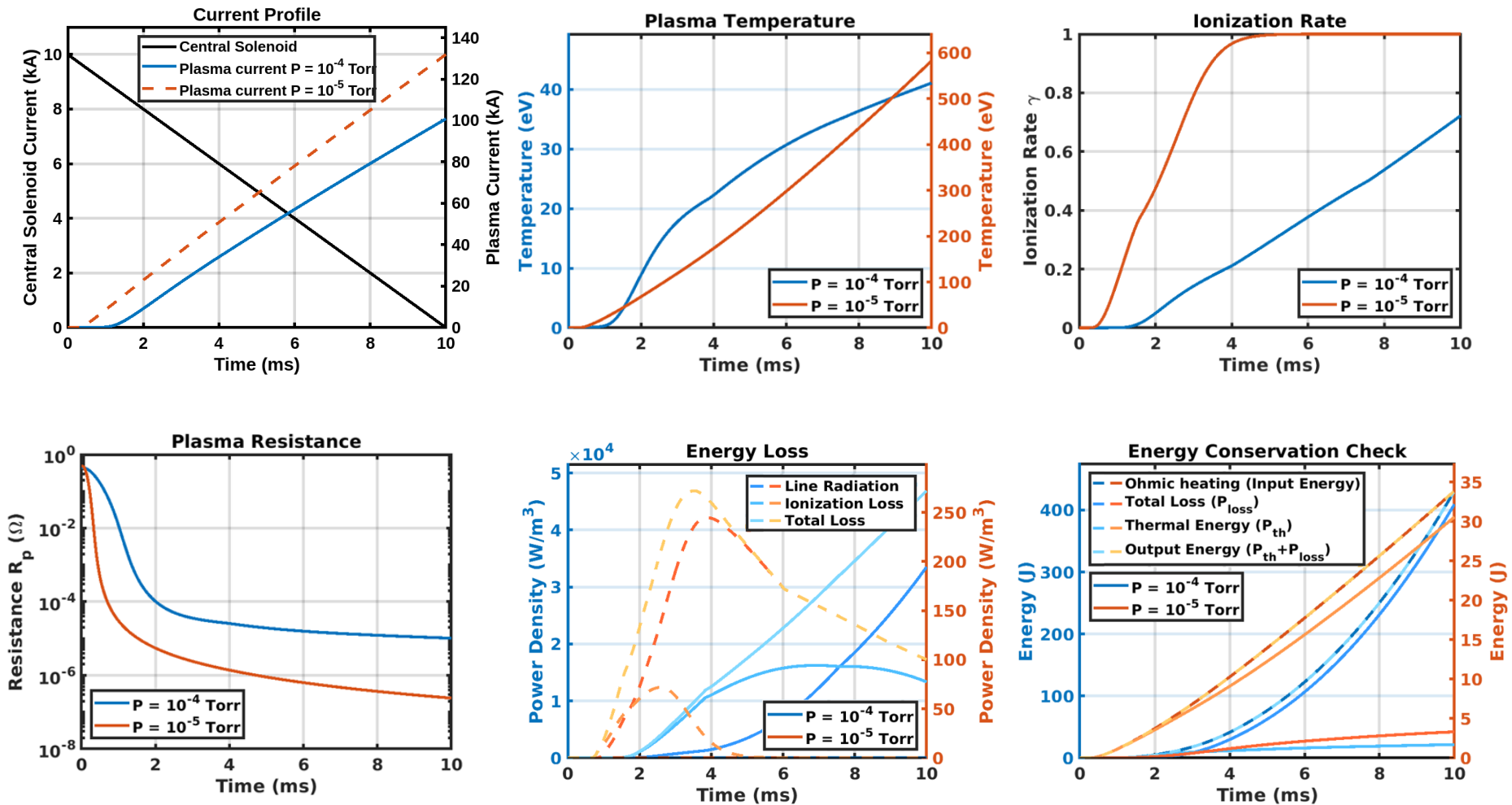
Eddy current on the chamber wall is induced when the current of the solenoid changes with time



Temperature of 100 eV is the threshold of radiation barrier by impurities



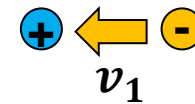
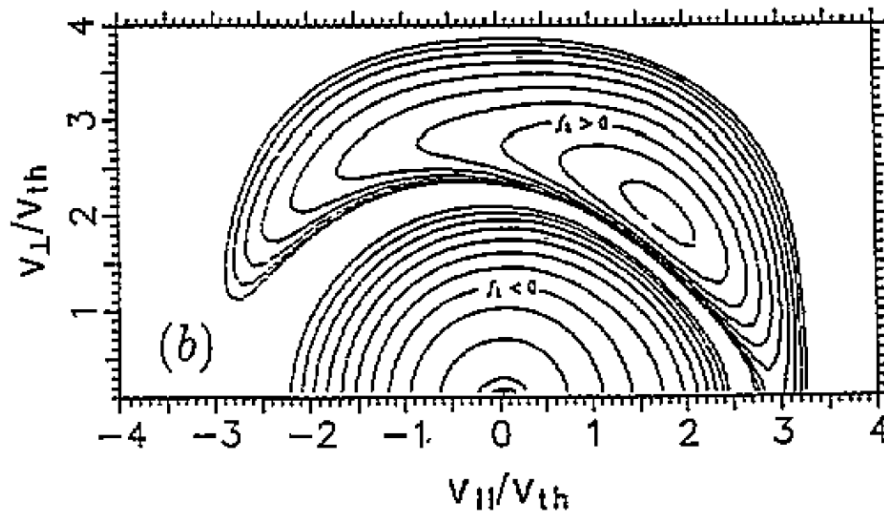
Plasma temperature can be estimated using the simple circuit model



The collisional re-distribution of the ECRH-driven anisotropy in E_{\perp} causes some parallel momentum to flow from e^{-} to ions

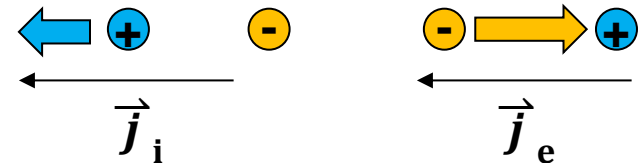


- Coulomb collisions are more efficient at lower energies.



Velocity: $v_2 > v_1$

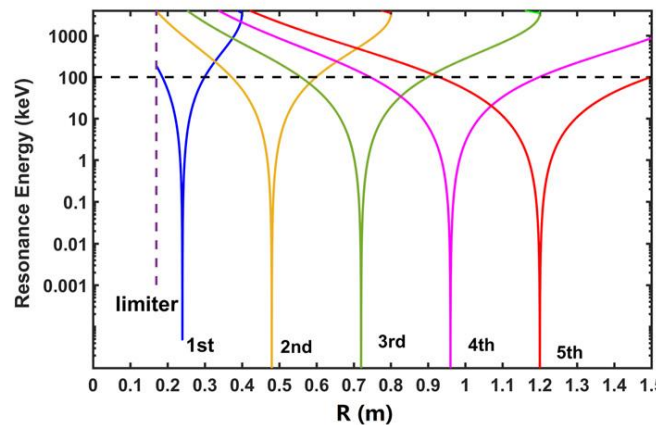
Collisions: $v_2 < v_1$



$$\vec{j}_p = -en_e \vec{v}_e + en_i \vec{v}_i$$

$$\vec{P} = n_e m_e \vec{v}_e + n_i m_i \vec{v}_i \approx 0$$

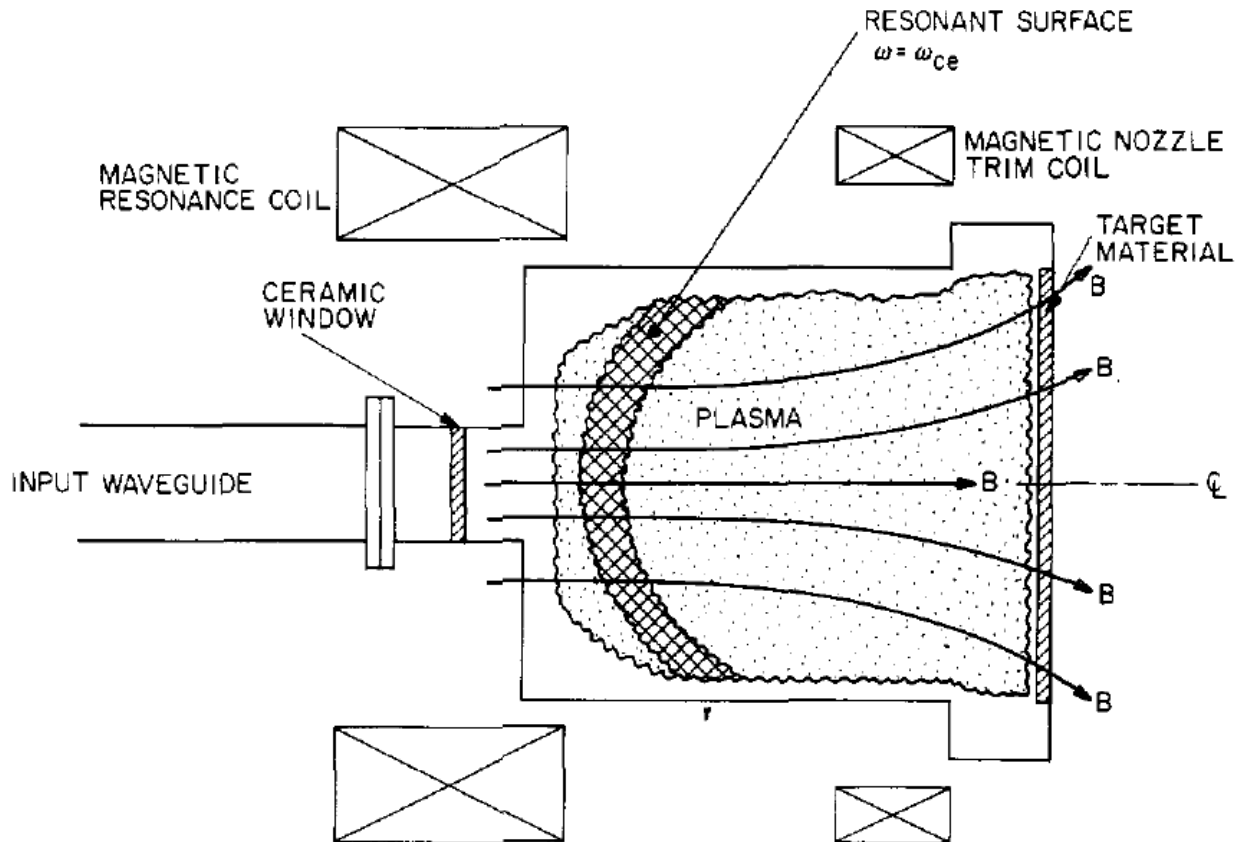
- Electron cyclotron current drive:



Strong absorption occurs when the frequency matches the electron cyclotron frequency



- Electron cyclotron resonance (ECR) plasma reactor



Electron cyclotron frequency depends on magnetic field only



$$m_e \frac{d\vec{v}}{dt} = -\frac{e}{c} \vec{v} \times \vec{B}$$

- Assuming $\vec{B} = B\hat{z}$ and the electron oscillates in x-y plane

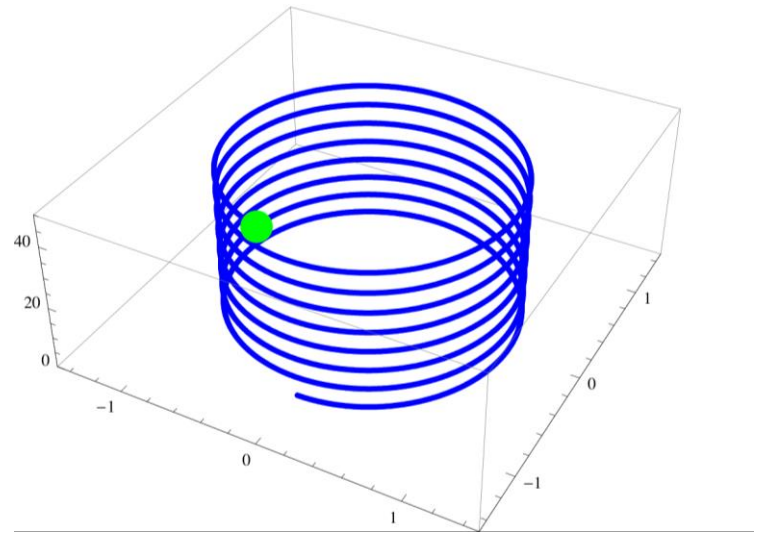
$$\begin{aligned} m_e \dot{v}_x &= -\frac{e}{c} B v_y & m_e \dot{v}_z &= 0 \\ m_e \dot{v}_y &= \frac{e}{c} B v_x \end{aligned}$$

$$\ddot{v}_x = -\frac{eB}{m_e c} \dot{v}_y = -\left(\frac{eB}{m_e c}\right)^2 v_x$$

$$\ddot{v}_y = -\frac{eB}{m_e c} \dot{v}_x = -\left(\frac{eB}{m_e c}\right)^2 v_y$$

- Therefore

$$\omega_{ce} = \frac{eB}{m_e c}$$



Electrons keep getting accelerated when a electric field rotates in electron's gyrofrequency



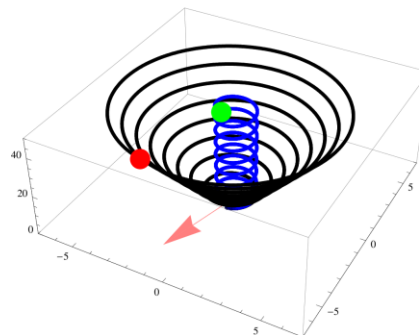
$$m_e \frac{d\vec{v}}{dt} = -\frac{e}{c} \vec{v} \times \vec{B} - e \vec{E} \quad \vec{B} = B_0 \hat{z} \quad \vec{E} = E_0 [\hat{x} \cos(\omega t) + \hat{y} \sin(\omega t)]$$

$$m_e \dot{v}_x = -\frac{e}{c} B v_y + E_0 \cos(\omega t) \quad m_e \dot{v}_y = \frac{e}{c} B v_x + E_0 \sin(\omega t) \quad m_e \dot{v}_z = 0$$

$$\ddot{v}_x = -\frac{eB}{m_e c} \dot{v}_y - \frac{E_0}{m_e} \omega \cos(\omega t) = -\omega_{ce}^2 v_x - \frac{E_0}{m_e} (\omega_{ce} + \omega) \cos(\omega t)$$

$$\ddot{v}_y = -\frac{eB}{m_e c} \dot{v}_x + \frac{E_0}{m_e} \omega \sin(\omega t) = -\omega_{ce}^2 v_y + \frac{E_0}{m_e} (\omega_{ce} + \omega) \sin(\omega t)$$

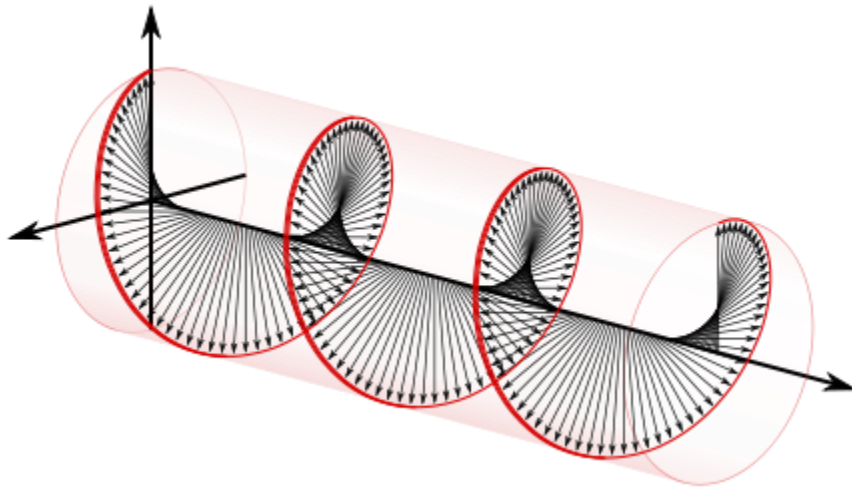
$$\omega_{ce} = \frac{eB}{m_e c}$$



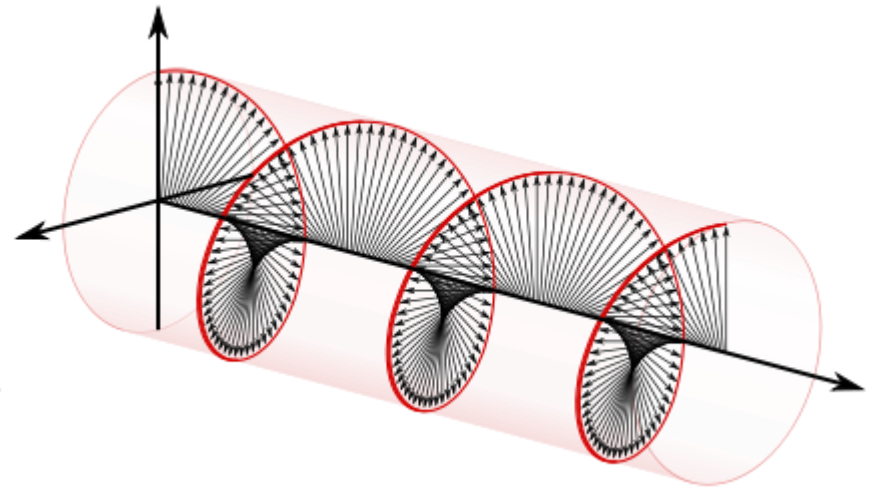
Electric field in a circular polarized electromagnetic wave keeps rotating as the wave propagates



- Right-handed polarization



- Left-handed polarization



Only right-handed polarization can resonance with electron's gyromotion

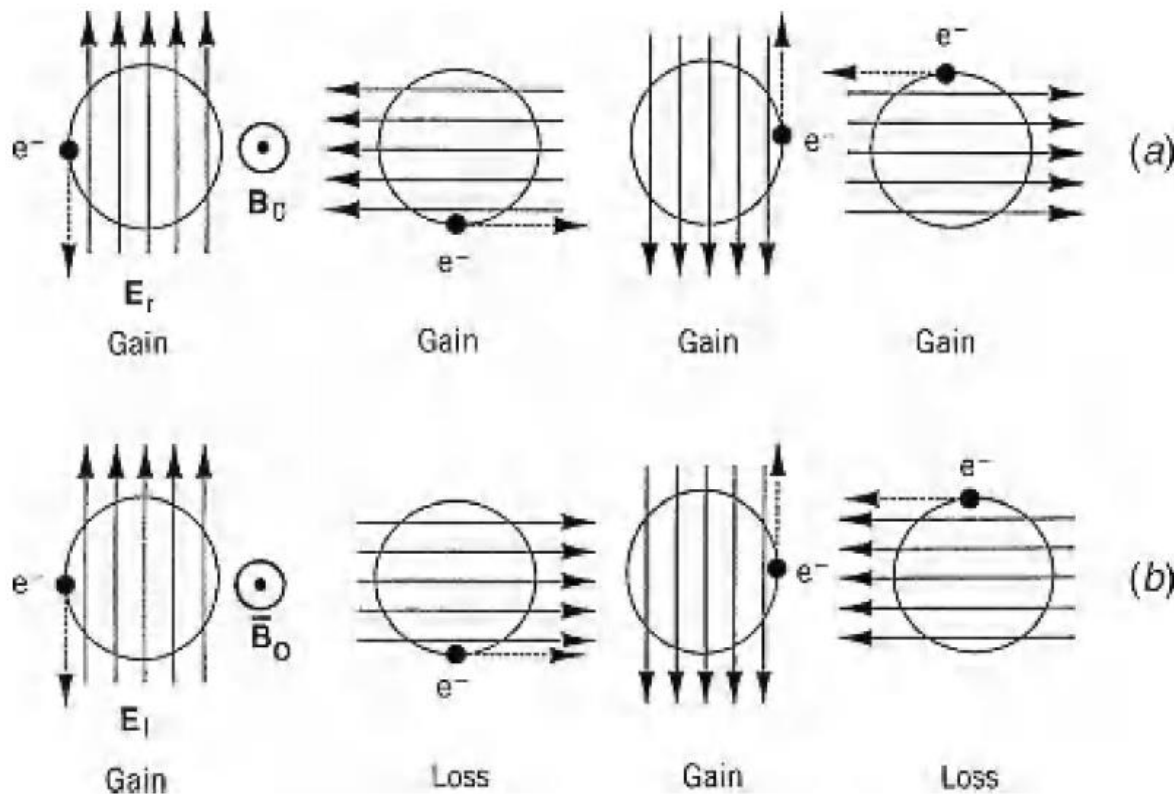
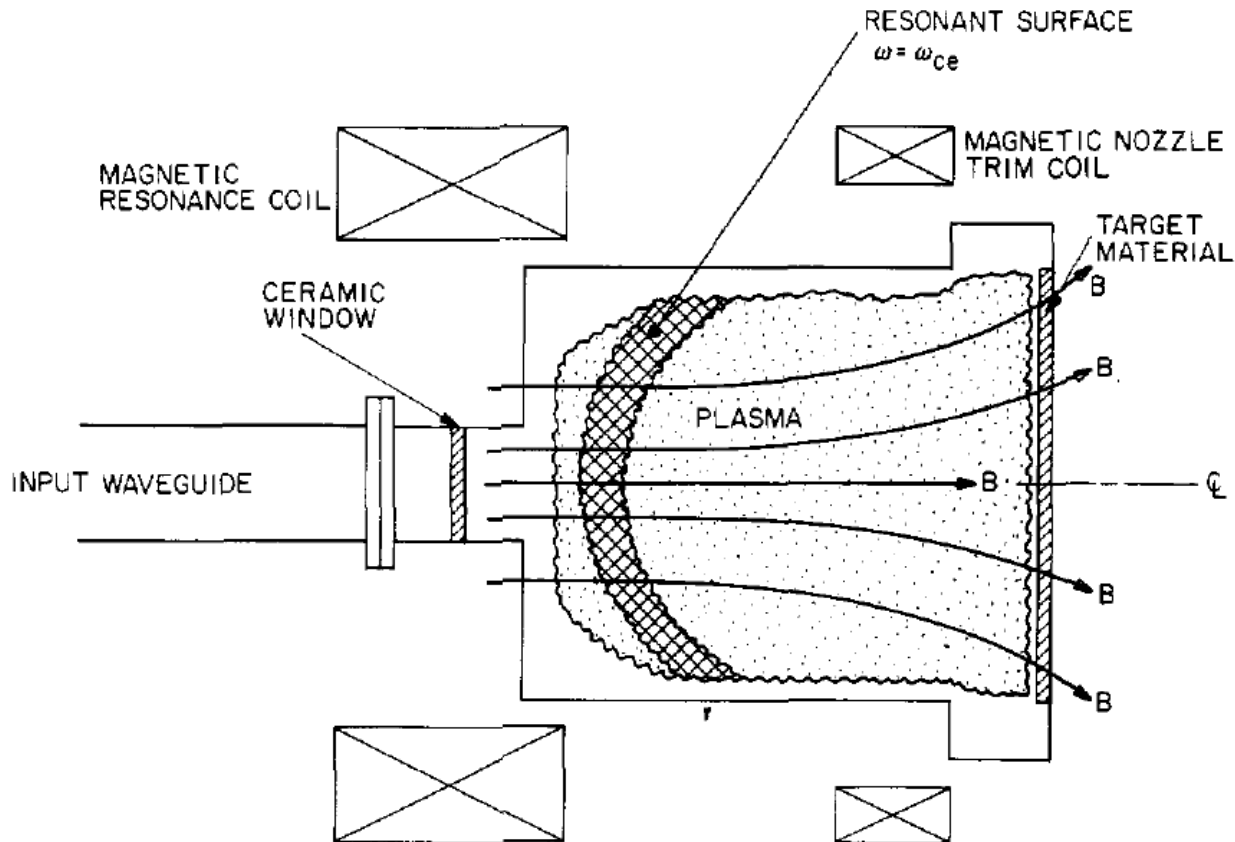


FIGURE 13.5. Basic principle of ECR heating: (a) continuous energy gain for right-hand polarization; (b) oscillating energy for left-hand polarization (after Lieberman and Gottscho, 1994).

Strong absorption occurs when the frequency matches the electron cyclotron frequency



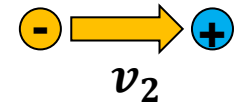
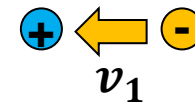
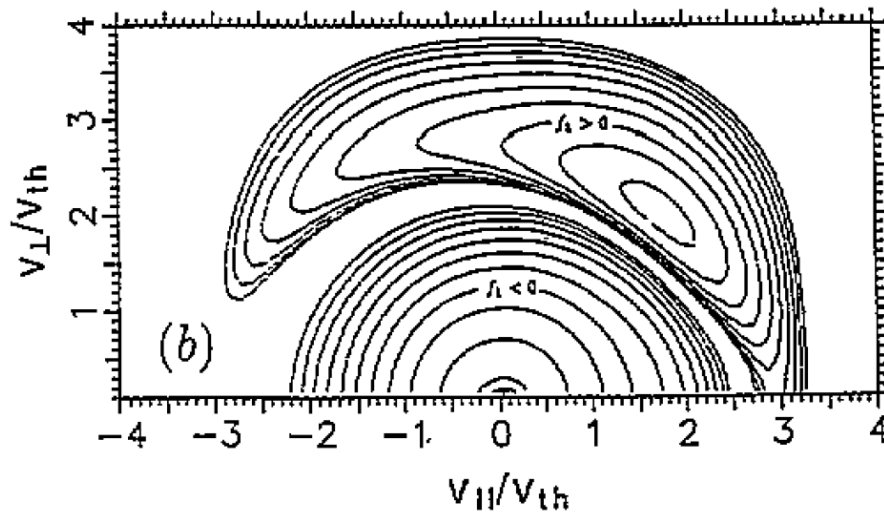
- Electron cyclotron resonance (ECR) plasma reactor



The collisional re-distribution of the ECRH-driven anisotropy in E_{\perp} causes some parallel momentum to flow from e^{-} to ions

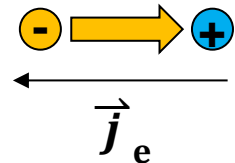
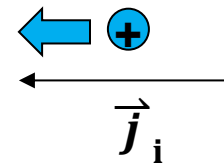


- Coulomb collisions are more efficient at lower energies.



Velocity: $v_2 > v_1$

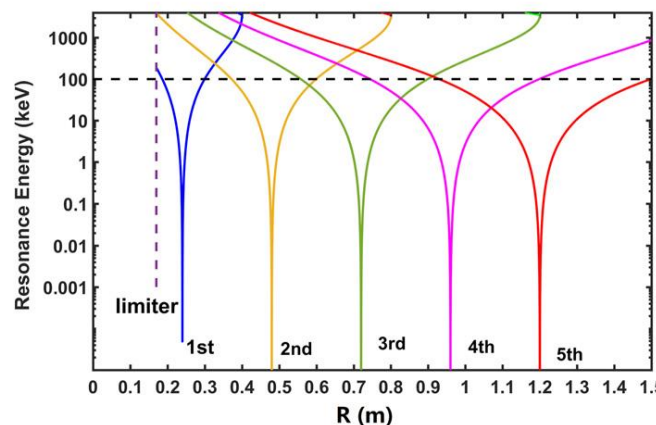
Collisions: $v_2 < v_1$



$$\vec{j}_p = -en_e \vec{v}_e + en_i \vec{v}_i$$

$$\vec{P} = n_e m_e \vec{v}_e + n_i m_i \vec{v}_i \approx 0$$

- Electron cyclotron current drive:



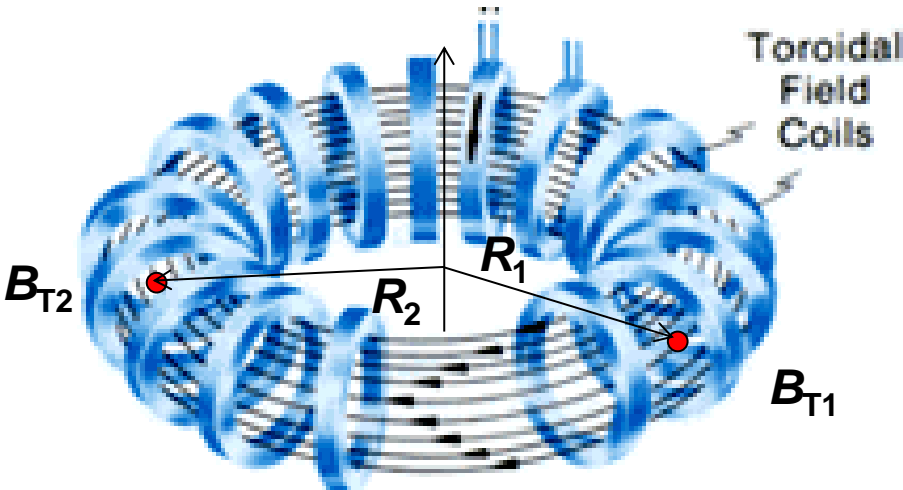
With poloidal fields, charged particles see nonuniform toroidal magnetic field



- W/o poloidal field

$$R_1 = R_2$$

$$B_{T1} = B_{T2}$$



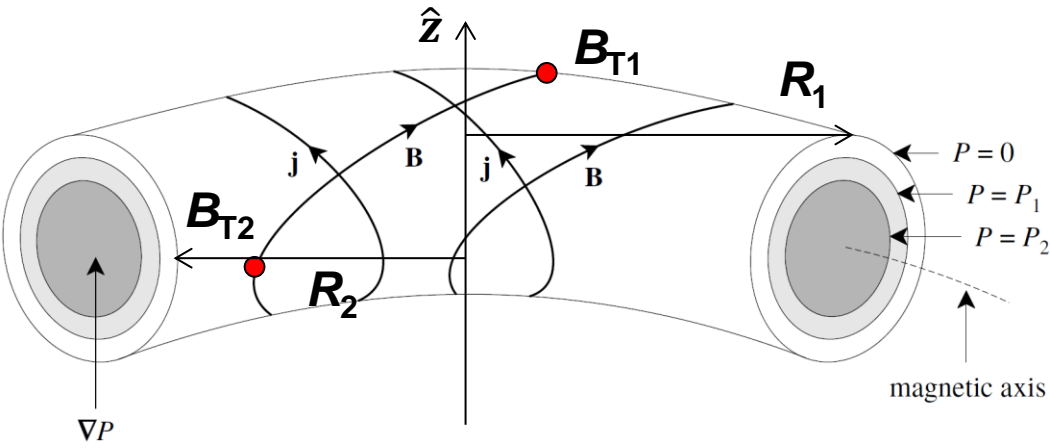
$$B_T \propto \frac{1}{R}$$

$$B_T \gg B_p$$

- W/ poloidal field

$$R_1 > R_2$$

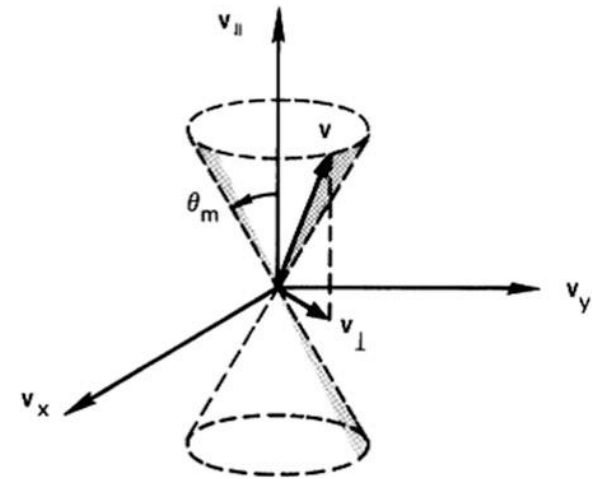
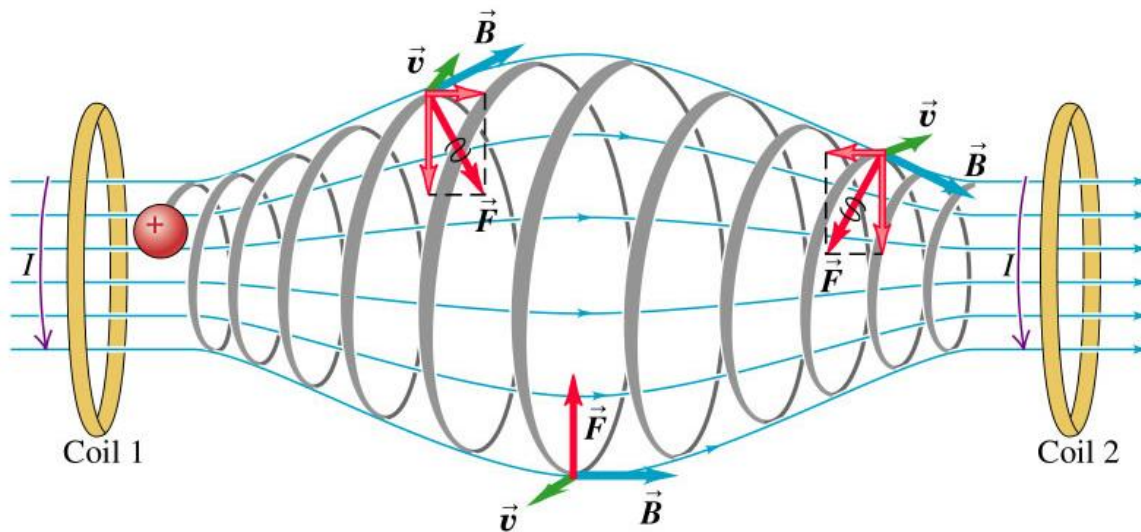
$$B_{T1} < B_{T2}$$



Charged particles can be partially confined by a magnetic mirror machine



- Charged particles with small v_{\parallel} eventually stop and are reflected while those with large v_{\parallel} escape.



$$\frac{1}{2}mv^2 = \frac{1}{2}mv_{\parallel}^2 + \frac{1}{2}mv_{\perp}^2 \quad \text{Invariant: } \mu \equiv \frac{1}{2} \frac{mv_{\perp}^2}{B}$$

$$v_{\perp}^{\prime 2} = v_{\perp o}^2 + v_{\parallel o}^2 \equiv v_o^2$$

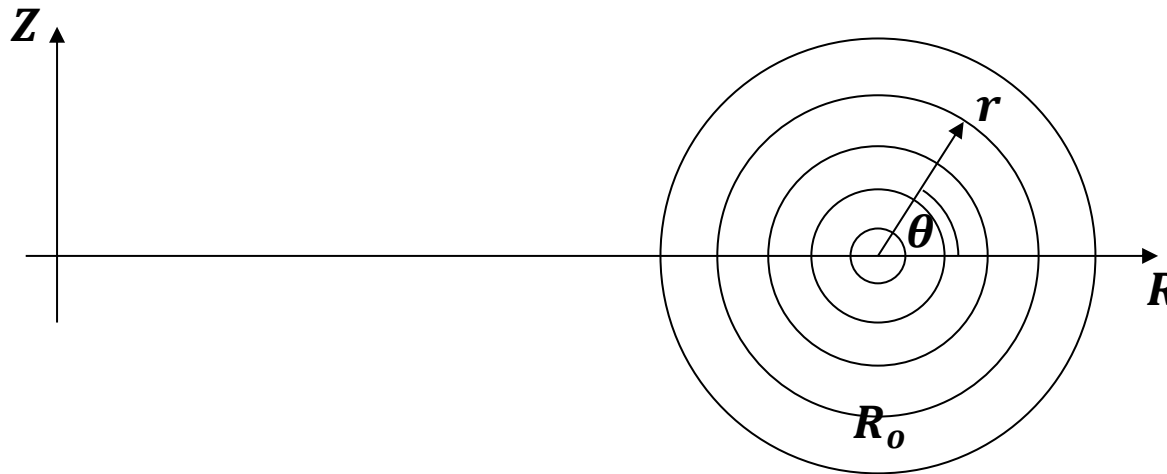
$$\frac{B_o}{B'} = \frac{v_{\perp o}^2}{v_{\perp}^{\prime 2}} = \frac{v_{\perp o}^2}{v_o^2} \equiv \sin^2 \theta$$

$$\frac{B_o}{B_m} \equiv \frac{1}{R_m} = \sin^2 \theta_m$$

- Large v_{\parallel} may occur from collisions between particles.

Those confined charged particle are eventually lost due to collisions.

Parallel velocity changes when particles follow field the field line



$$R \gg r$$

$$B_T \gg B_p$$

$$R = R_0 + r \cos \theta = R_0(1 + \epsilon \cos \theta)$$

Inverse aspect ratio:

$$\epsilon \equiv \frac{r}{R_0}$$

$$B \simeq \frac{B_0}{1 + \epsilon \cos \theta} \simeq B_0(1 - \epsilon \cos \theta)$$

Invariant: $\mu \equiv \frac{1}{2} \frac{mv_{\perp}^2}{B}$

$$\frac{v_{\perp}^2}{B_0(1 - \epsilon \cos \theta)} = \frac{v_{\perp 0}^2}{B_0(1 - \epsilon)}$$

$$v_{\perp}^2 = \frac{v_{\perp 0}^2(1 - \epsilon \cos \theta)}{1 - \epsilon}$$

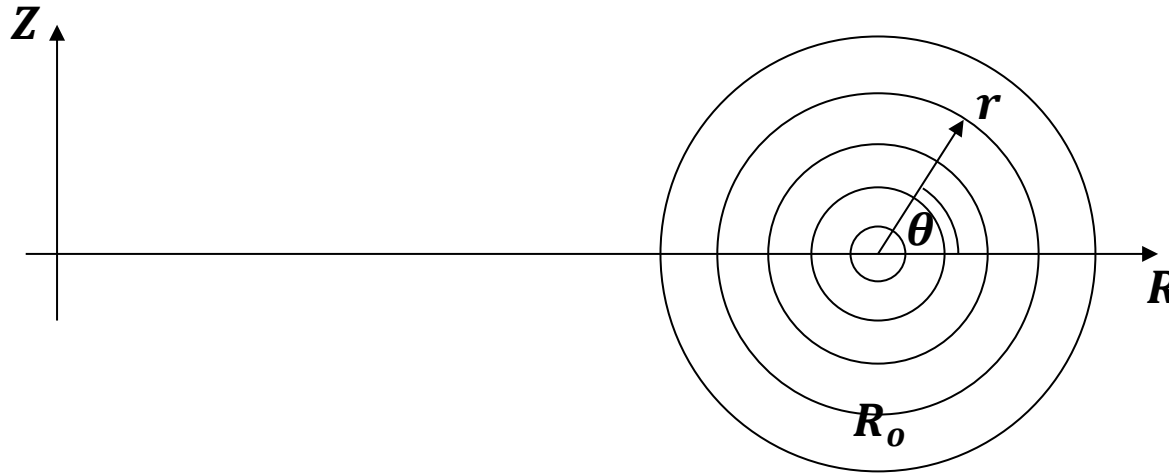
$$v^2 = v_{\perp}^2 + v_{\parallel}^2 = v_{\perp 0}^2 + v_{\parallel 0}^2$$

$$v_{\parallel}^2 = v^2 \left(1 - \frac{v_{\perp}^2}{v^2} \right)$$

$$v_{\parallel}^2 = v^2 \left[1 - \frac{v_{\perp 0}^2}{v^2} \frac{(1 - \epsilon \cos \theta)}{1 - \epsilon} \right]$$

$$\approx v^2 \left\{ 1 - \frac{v_{\perp 0}^2}{v^2} \left[1 + 2\epsilon \sin^2 \left(\frac{\theta}{2} \right) \right] \right\}$$

Particles may be trapped by nonuniform magnetic field



$$R \gg r$$
$$B_T \gg B_p$$

$$\epsilon \equiv \frac{r}{R_0}$$

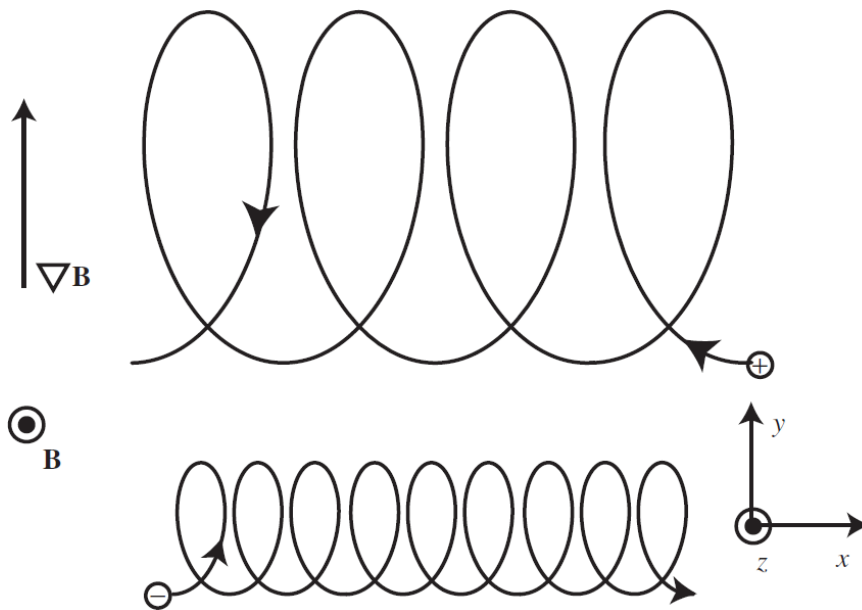
$$v_{||}^2 \approx v^2 \left\{ 1 - \frac{v_{\perp 0}^2}{v^2} \left[1 + 2\epsilon \sin^2 \left(\frac{\theta}{2} \right) \right] \right\}$$

- For $v_{||}^2 \geq 0$, particles are passing.
- For $v_{||}^2 \leq 0$, particles are trapped.

Charge particles drift across magnetic field lines when the magnetic field is not uniform or curved

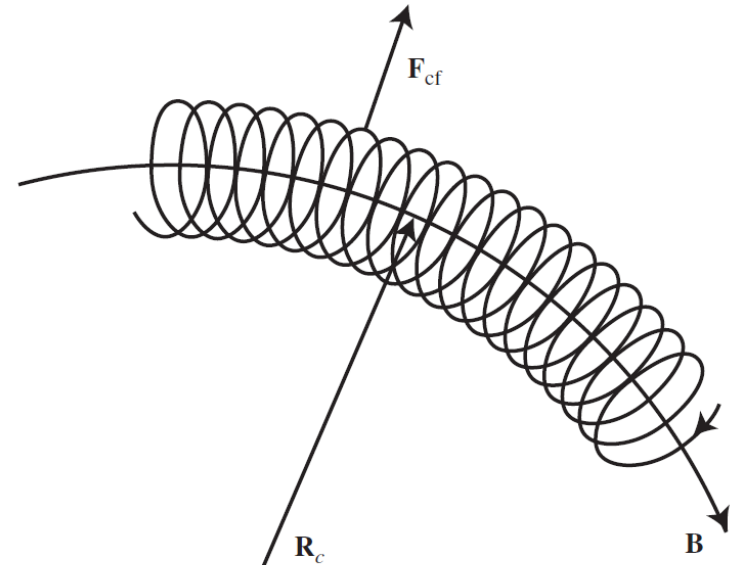


- Gradient-B drift



$$\vec{v}_{\nabla} = \frac{mv_{\perp}^2}{2q} \frac{\vec{B} \times \nabla B}{B^3}$$

- Curvature drift



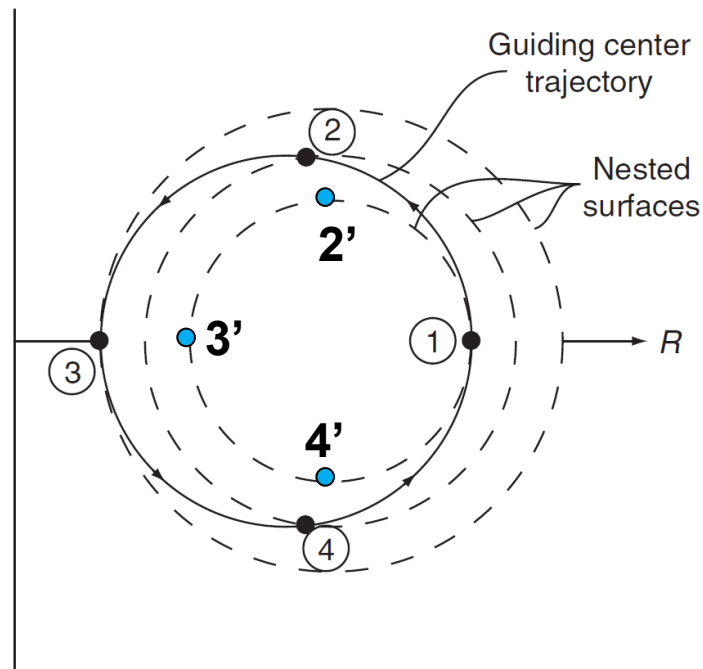
$$\vec{v}_R = \frac{mv_{\parallel}^2}{2q} \frac{\vec{R}_c \times \vec{B}}{R_c B^2}$$

$$\vec{v}_{\text{total}} = \vec{v}_R + \vec{v}_{\nabla} = \frac{\vec{B} \times \nabla B}{\omega_c B^2} \left(v_{\parallel}^2 + \frac{1}{2} v_{\perp}^2 \right) = \frac{m}{q} \frac{\vec{R}_c \times \vec{B}}{R_c^2 B^2} \left(v_{\parallel}^2 + \frac{1}{2} v_{\perp}^2 \right)$$

For passing particles, they drift back to the original position with a “semicircle” orbit



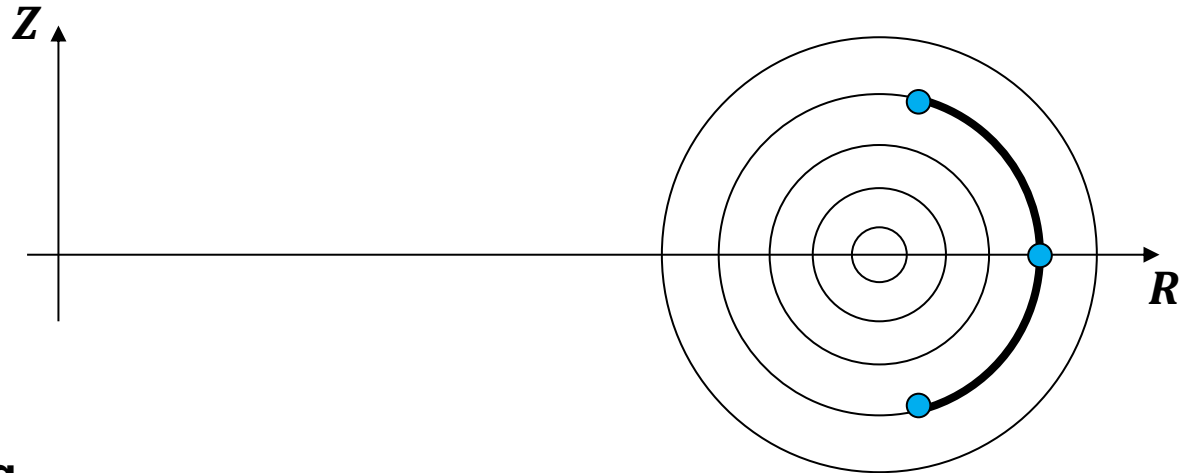
$$\vec{v}_{\text{total}} = \vec{v}_R + \vec{v}_\nabla = \frac{\vec{B} \times \nabla B}{\omega_c B^2} \left(v_{\parallel}^2 + \frac{1}{2} v_{\perp}^2 \right) = \frac{m}{q} \frac{\vec{R}_c \times \vec{B}}{R_c^2 B^2} \left(v_{\parallel}^2 + \frac{1}{2} v_{\perp}^2 \right)$$



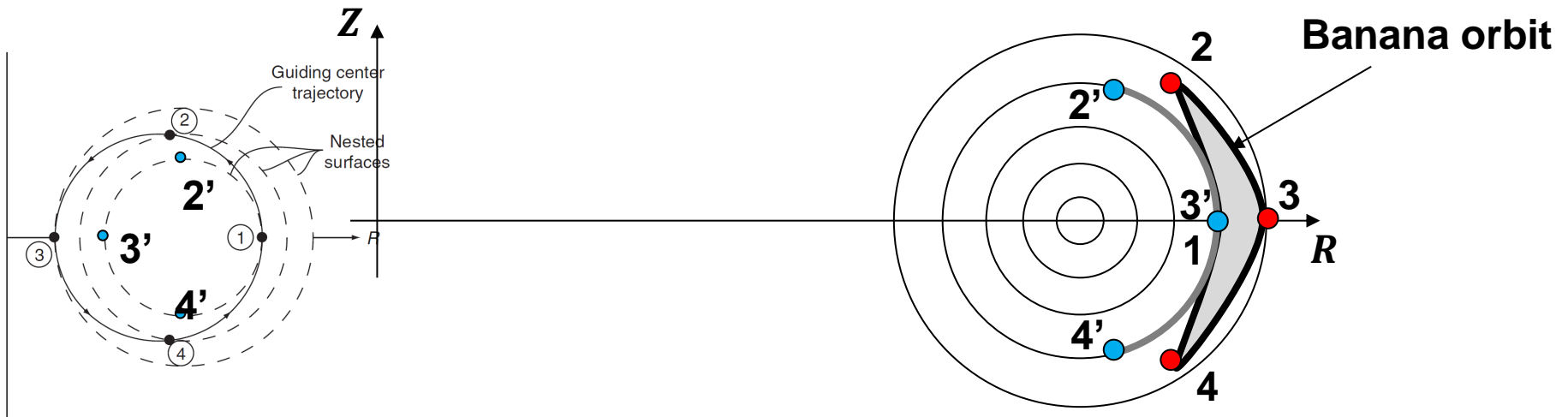
For trapped particles, they drift back to the original position with a banana orbit



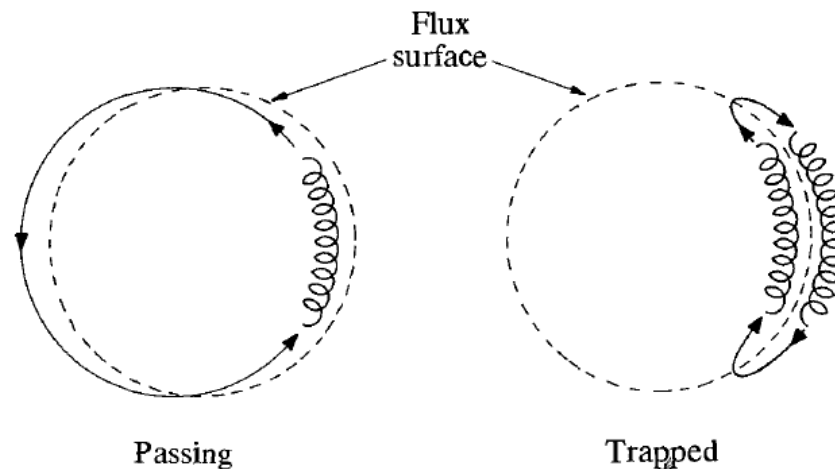
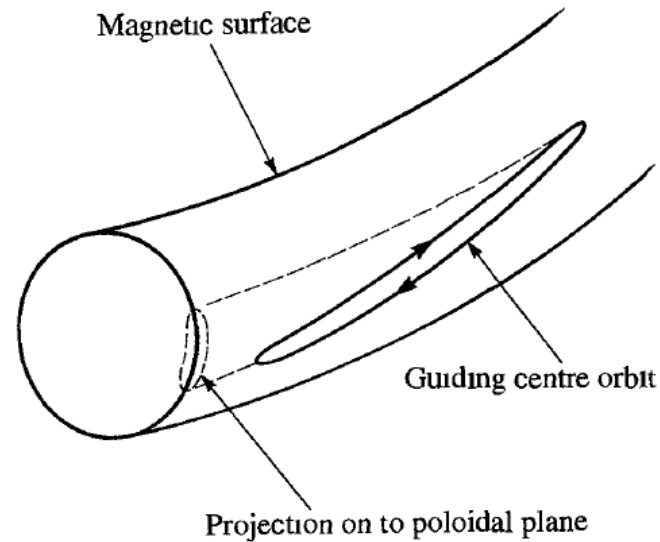
- W/o drifting



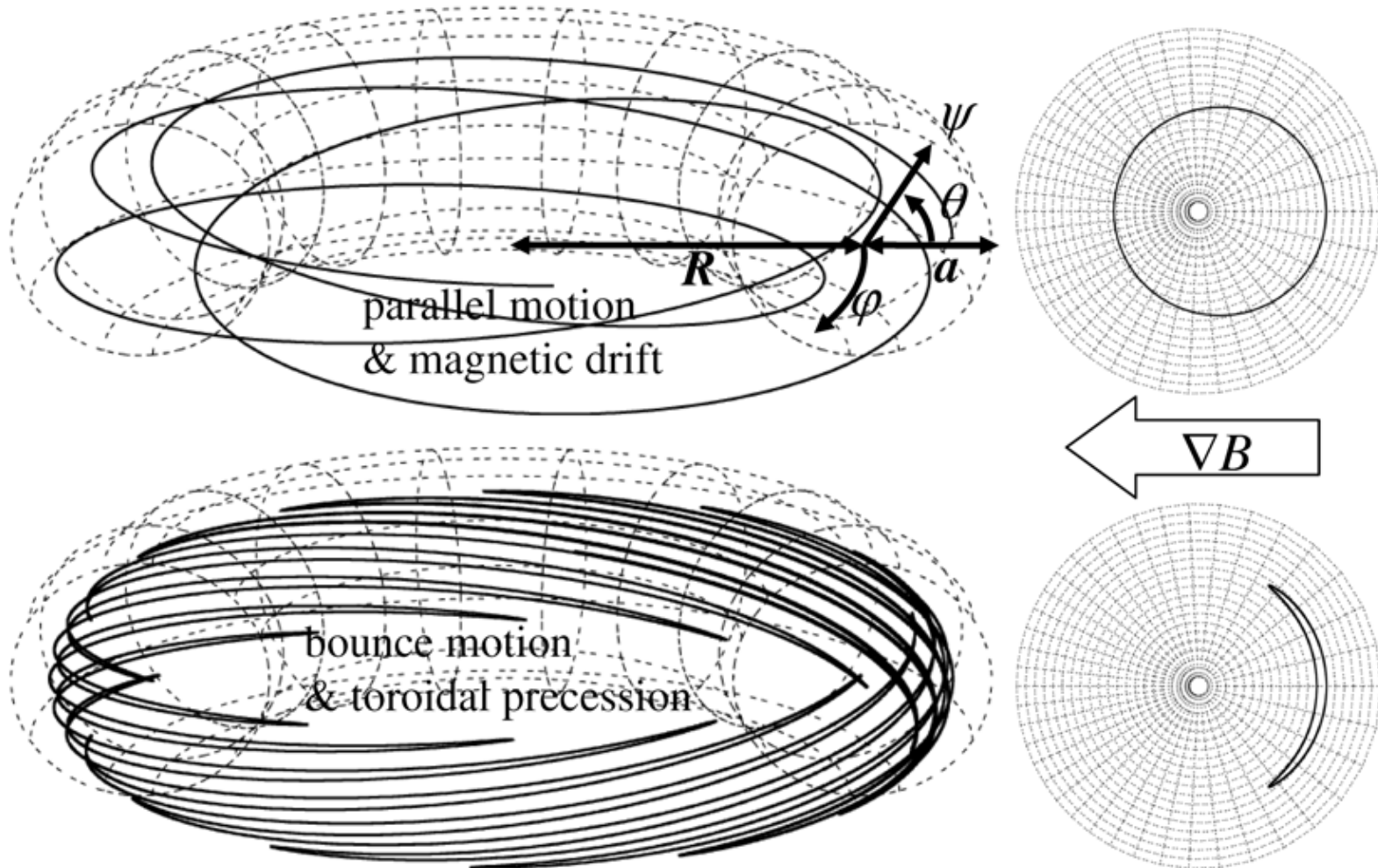
- W/ drifting



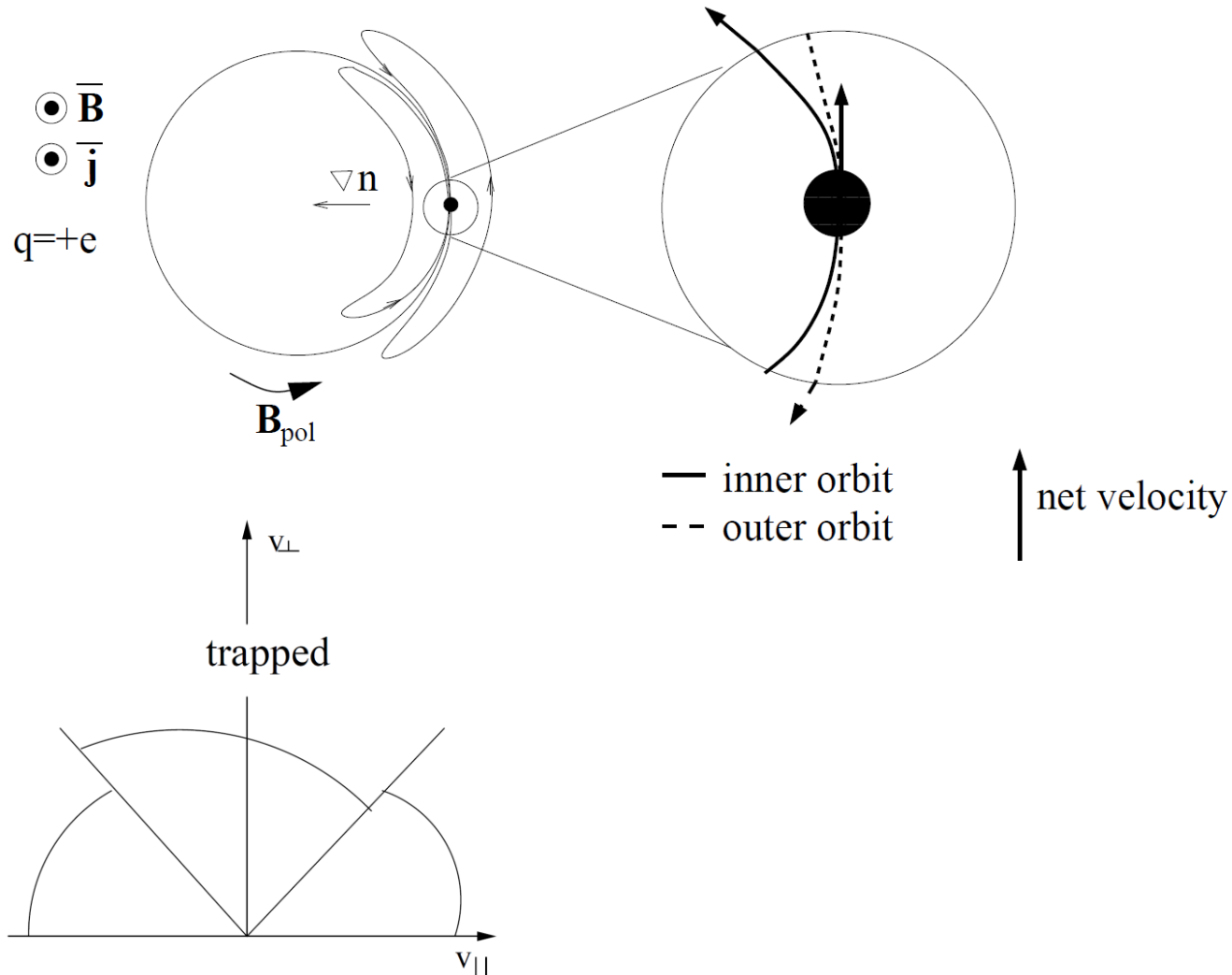
Trajectories of charged particles



The trajectories of charged particles follow the toroidal field lines



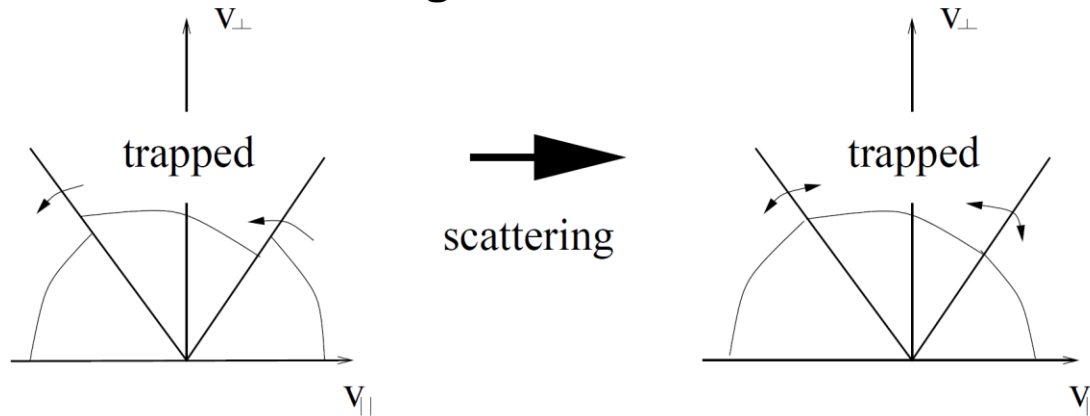
A banana current is generated when there is a pressure gradient in the plasma



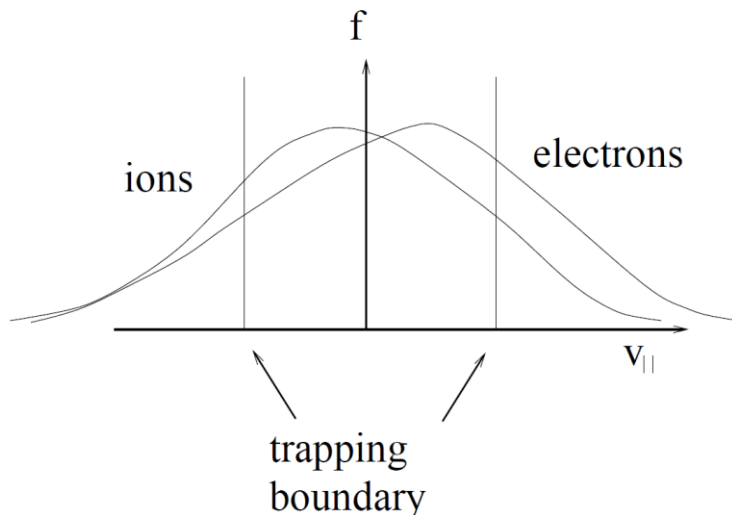
Bootstrap current is generated when passing particles are scattered by the trapped particles



- Scattering smooths the velocity distribution and shifts it in the parallel direction, i.e., a current is generated. It is called the bootstrap current.

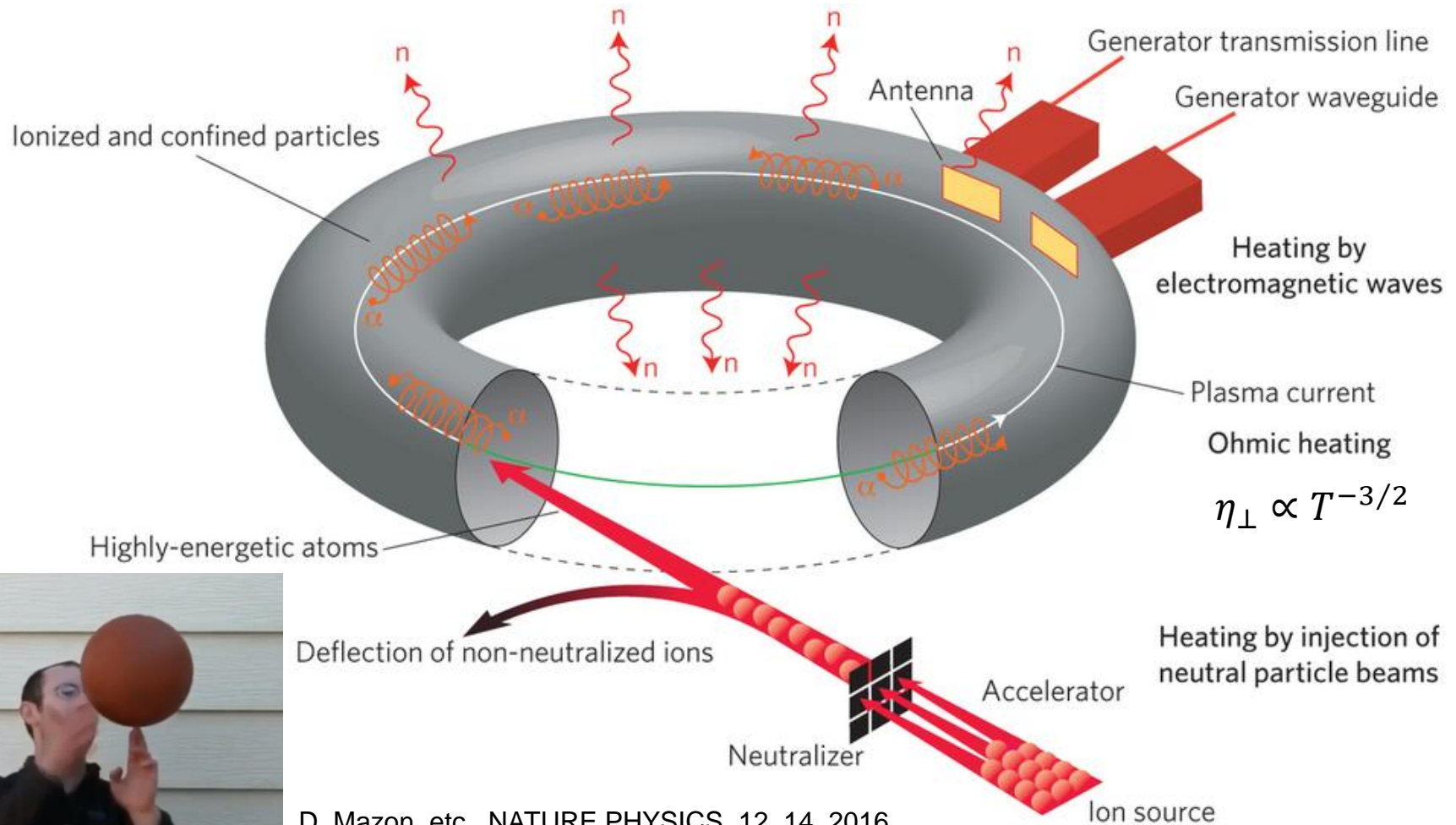


$$\mathbf{j} = -en\mathbf{u}_{||e} + en\mathbf{u}_{||i} = 4\epsilon^{3/2} \frac{1}{B_p} T \frac{dn}{dr}$$



- The bootstrap current is vital for steady-state operation.

Neutral beam injector is one of the main heat mechanisms in MCF



D. Mazon, etc., NATURE PHYSICS, 12, 14, 2016

<https://zh.wikihow.com/%E5%9C%A8%E6%89%8B%E6%8C%87%E4%B8%8A%E8%BD%AC%E7%AF%AE%E7%90%83>



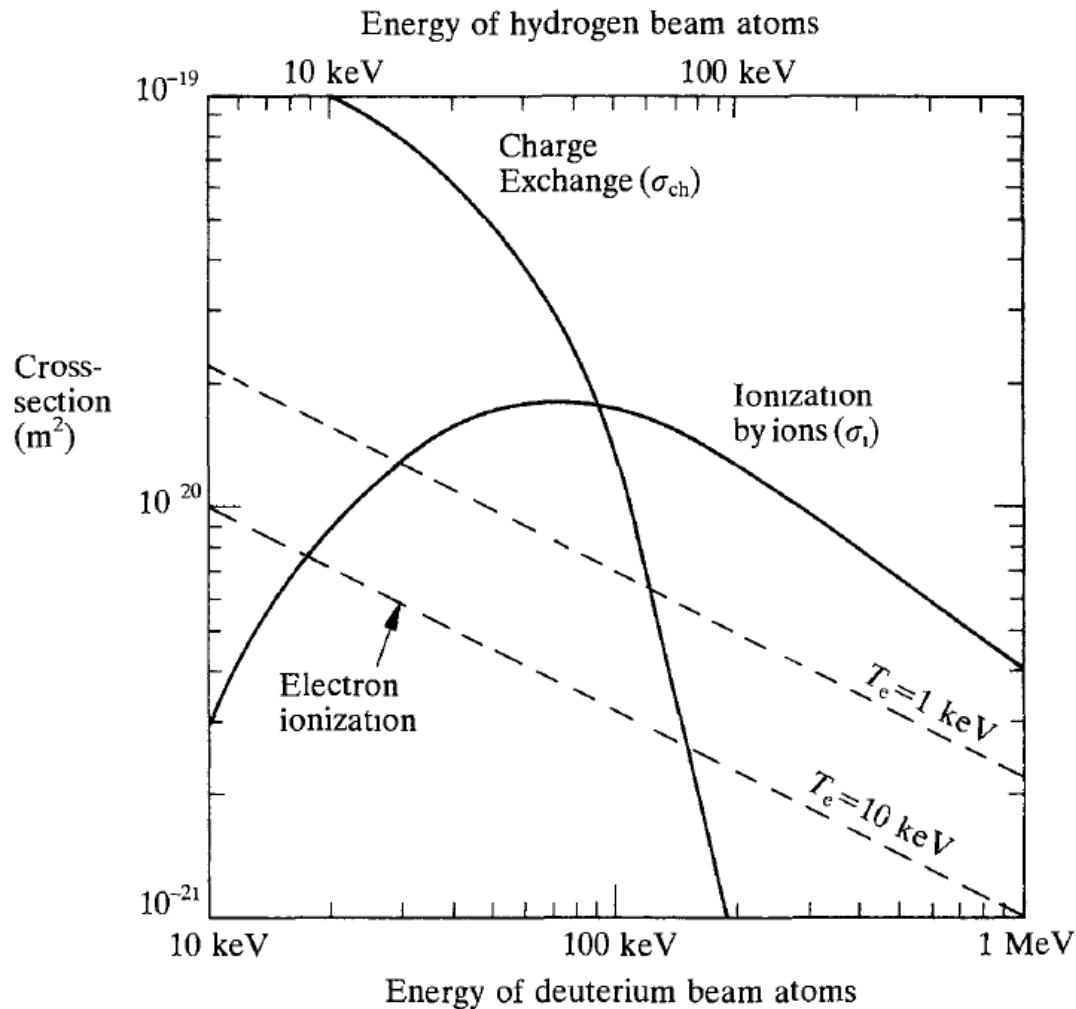
Varies way of heating a MCF device



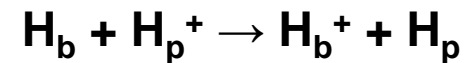
	System	Frequency/ energy	Maximum power coupled to plasma	Overall system efficiency	Development/ demonstration required	Remarks
ECRF	Demonstrated in tokamaks	28–157 GHz	2.8 MW, 0.2 s	30–40%	Power sources and windows, off-axis CD	Provides off-axis CD
	ITER needs	150–170 GHz	50 MW, SS			
ICRF	Demonstrated in tokamaks	25–120 MHz	22 MW, 3 s (L-mode); 16.5 MW, 3 s (H-mode)	50–60%	ELM tolerant system	Provides ion heating and smaller ELMs
	ITER needs	40–75 MHz	50 MW, SS			
LHRF	Demonstrated in tokamaks	1.3–8 GHz	2.5 MW, 120 s; 10 MW, 0.5 s	45–55%	Launcher, coupling to H-mode	Provides off-axis CD
	ITER needs	5 GHz	50 MW, SS			
NBI	+ve ion Demonstrated in tokamaks	80–140 keV	40 MW, 2 s; 20 MW, 8 s	35–45%	None	Not applicable
	ITER needs	None	None			
	–ve ion Demonstrated in tokamaks	0.35 MeV	5.2 MW, D ⁺ , 0.8 s (from 2 sources)	~37%	System, tests on tokamak, plasma CD	provides rotation
	ITER needs	1 MeV	50 MW, SS			

‘SS’ indicates steady state

Neutral atoms are ionized by collisions in the plasma



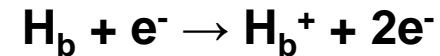
- Charge exchange:



- Ionization by ions



- Ionization by electrons

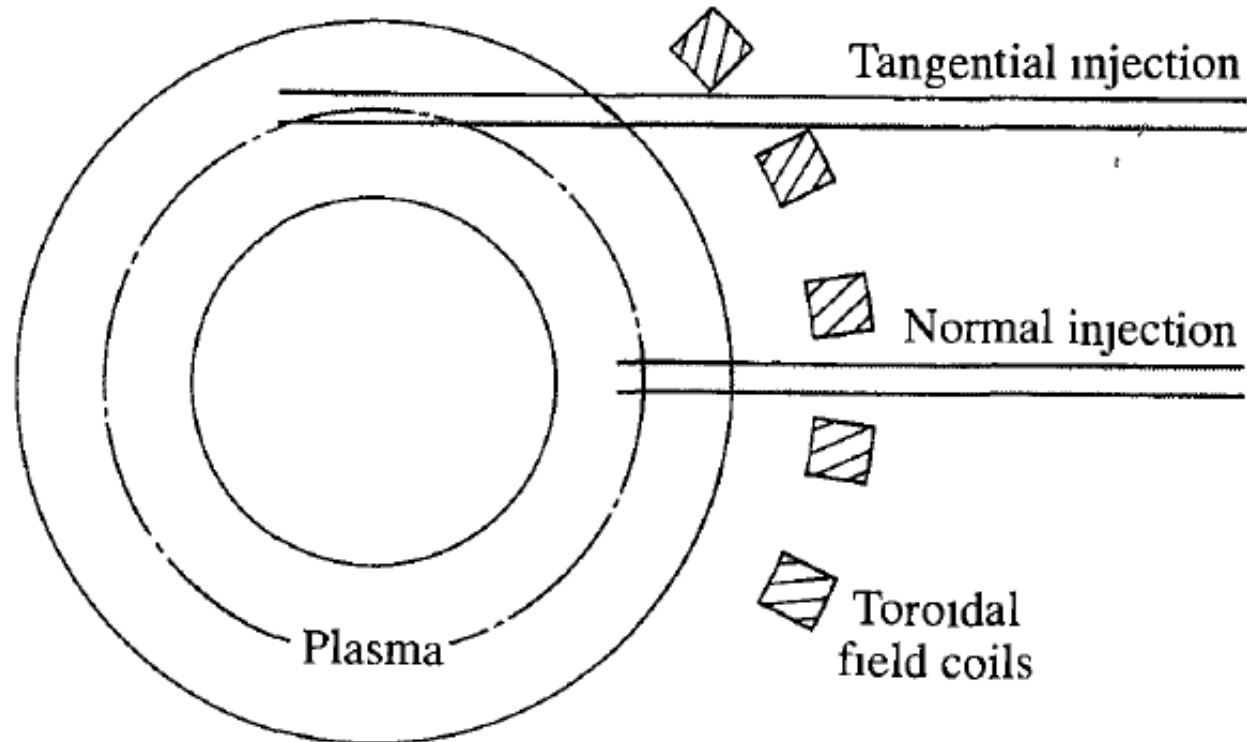


b: beam
p: plasma

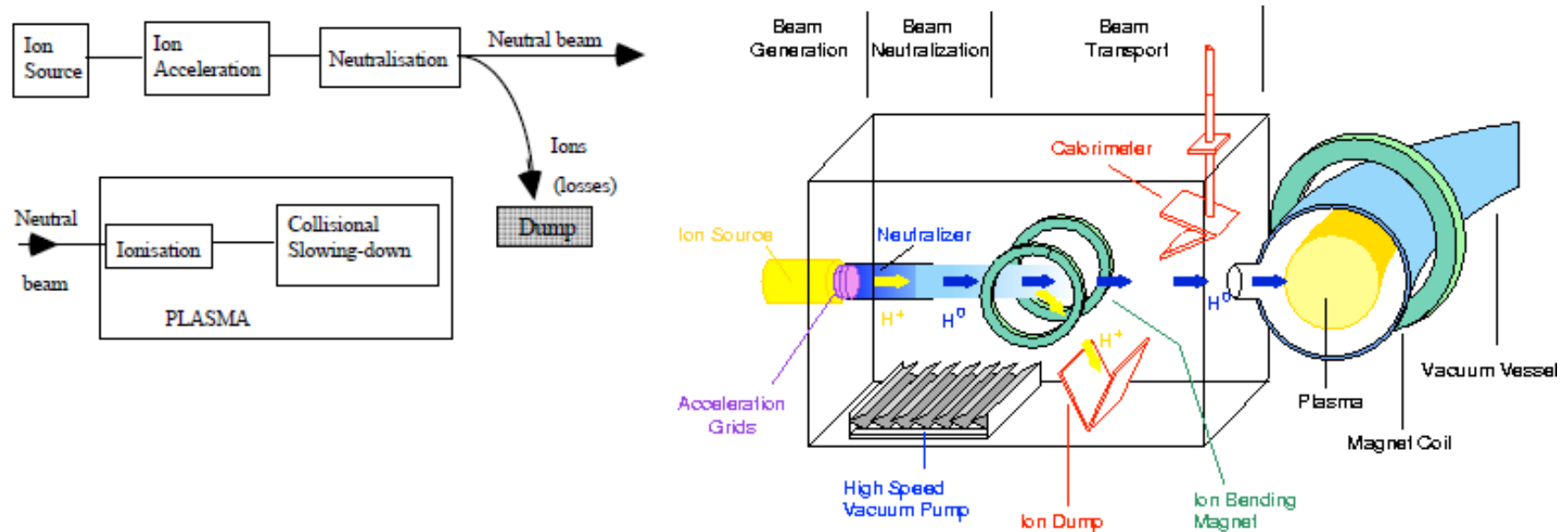
Neutral beam absorption length increases with tangential injection



- It is more difficult to access through the toroidal field coils with tangential injection.

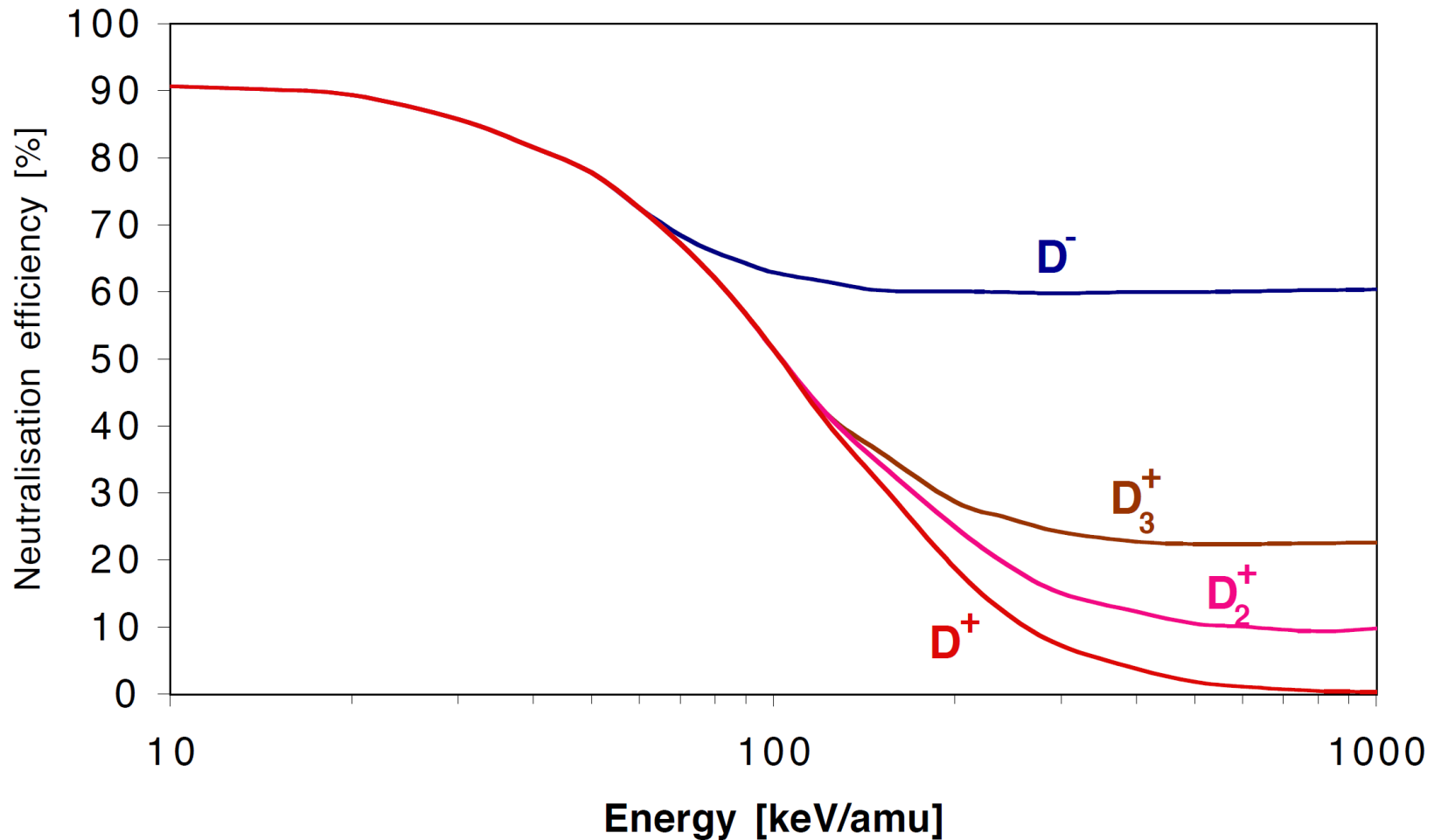


Neutral particles heat the plasma via coulomb collisions



1. create energetic (fast) neutral ions
2. ionize the neutral particles
3. heat the plasma (electrons and ions) via Coulomb collisions

Negative ion source is preferred due to higher neutralization efficiency

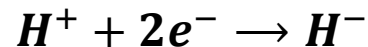
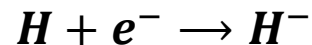
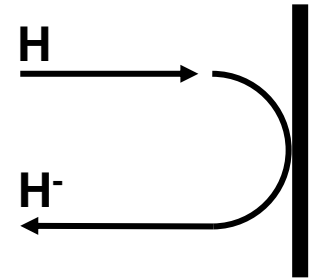


There are two ways to make negative ions – surface and volume production

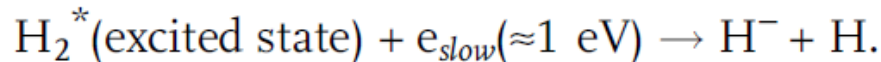
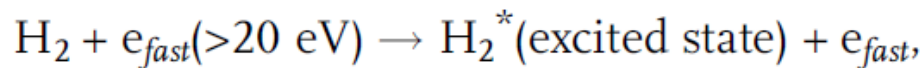


- **Surface production, depends on :**

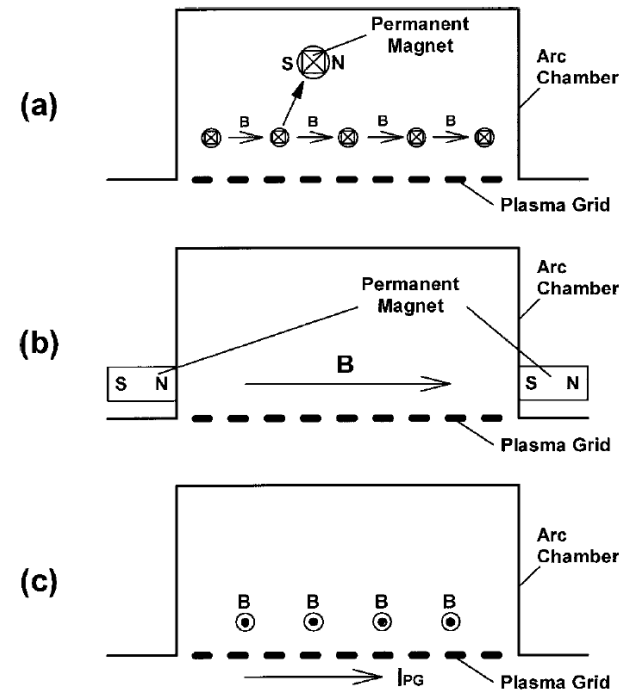
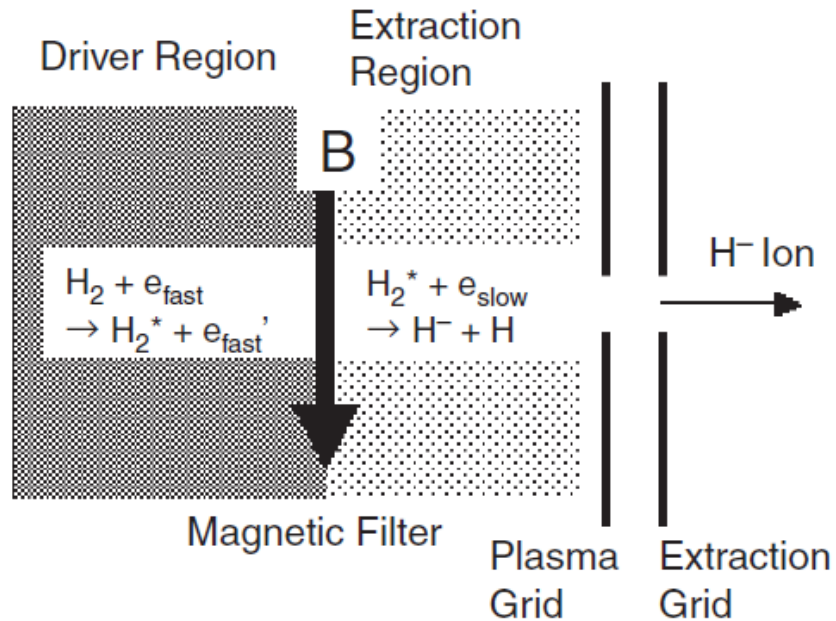
- Work function Φ
- Electron affinity level, 0.75 eV for H^-
- Perpendicular velocity
- Work function can be reduced by covering the metal surface with cesium



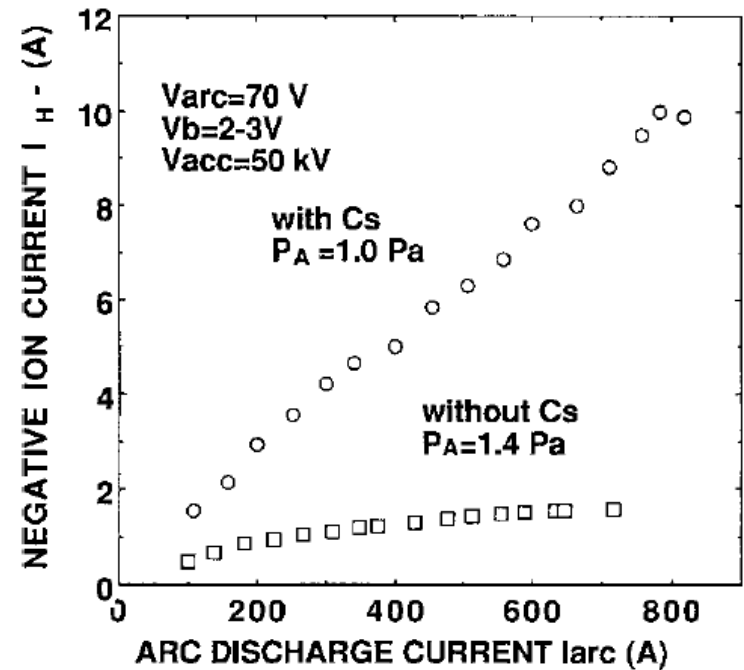
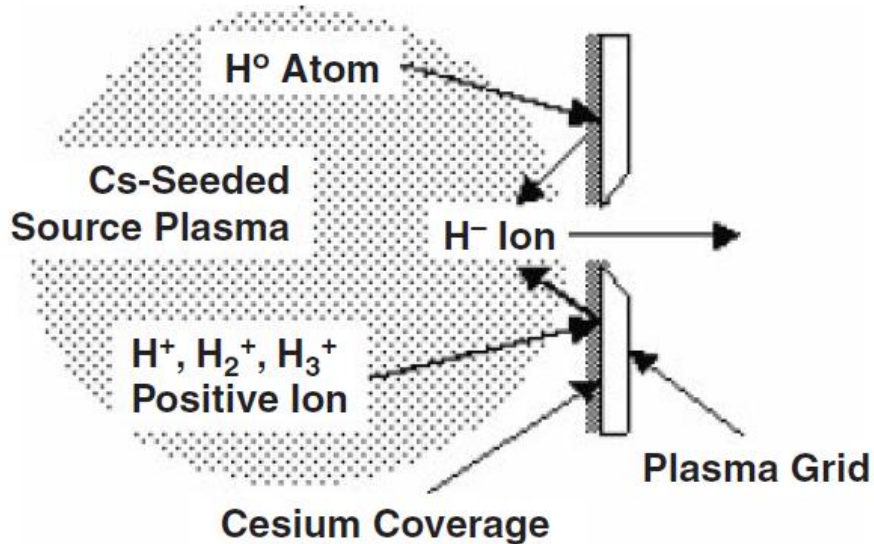
- **Volume production:**



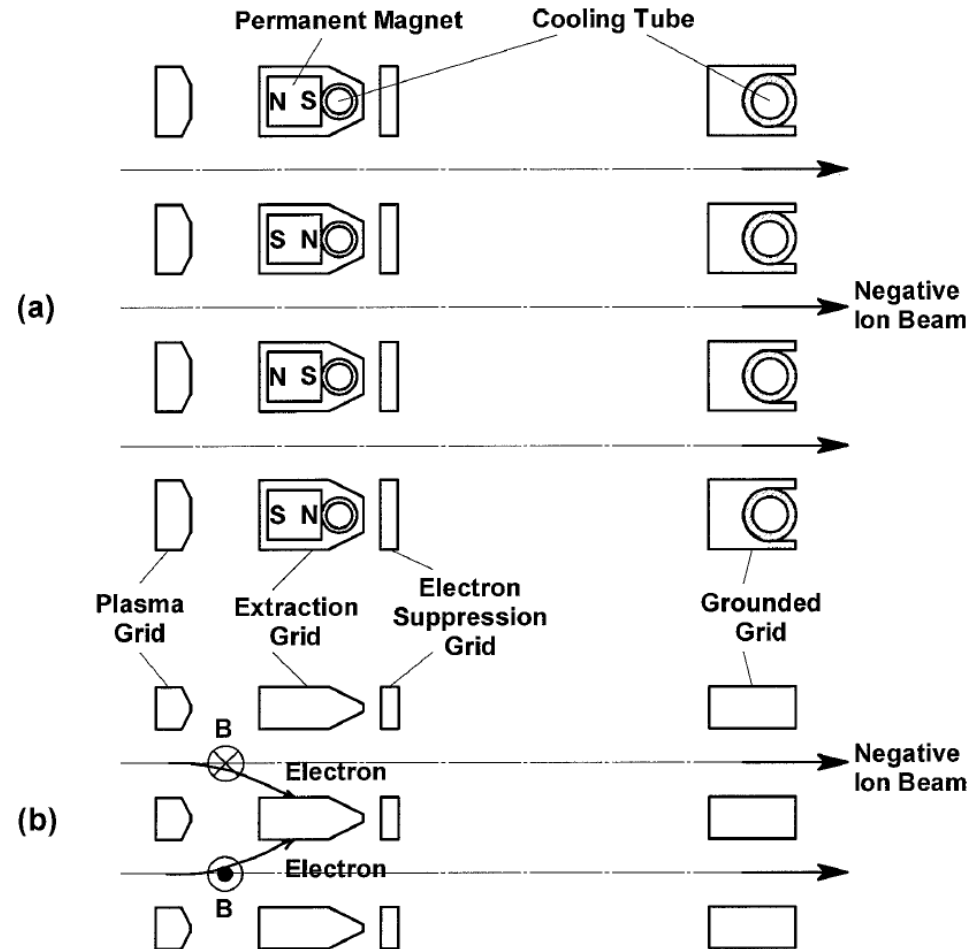
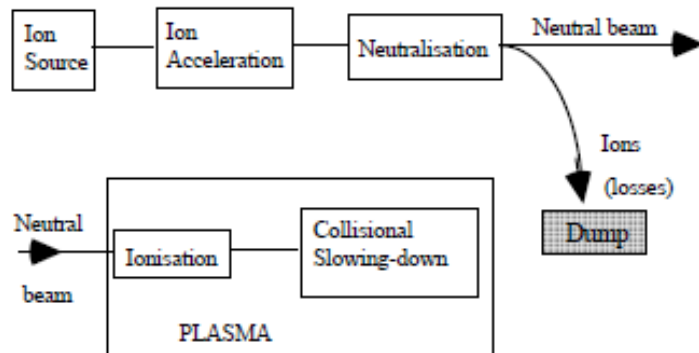
Two-chamber method of negative ions in volume production with a magnetic filter



Adding cesium increases negative ion current



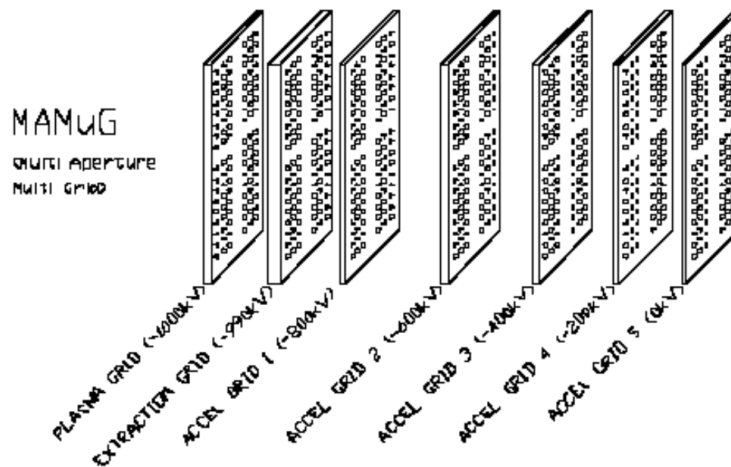
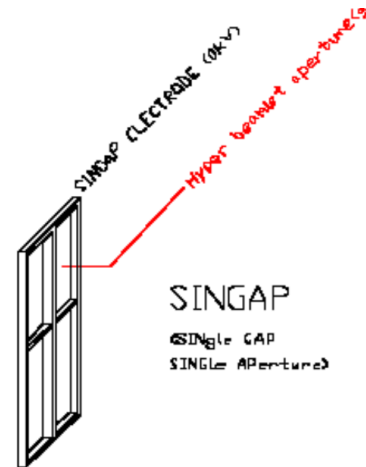
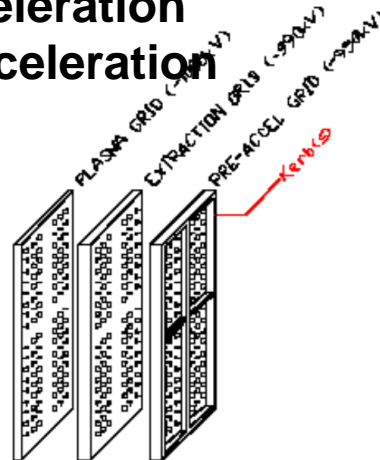
Electrons need to be filtered out since they are extracted together with negative ions



Acceleration

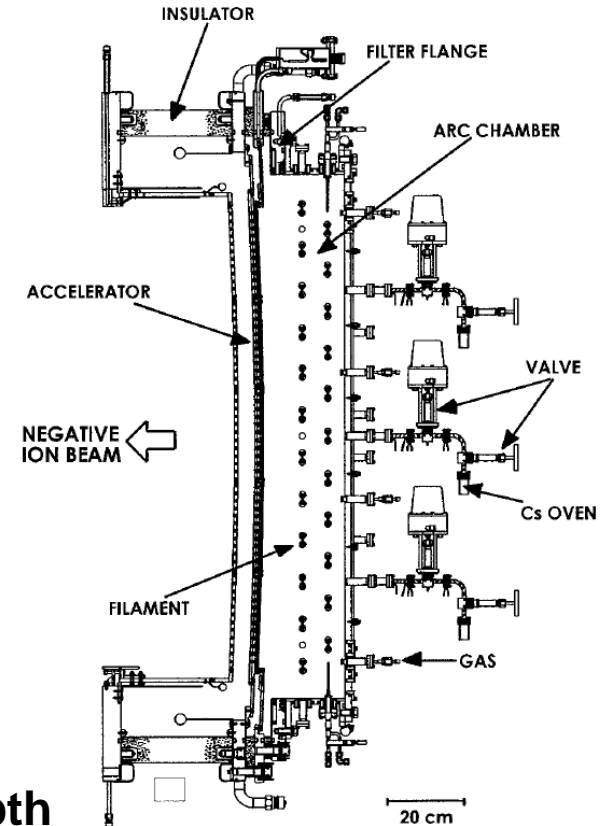
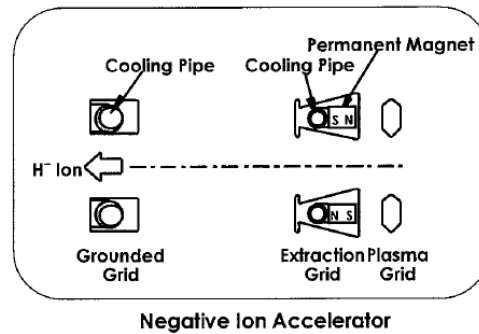
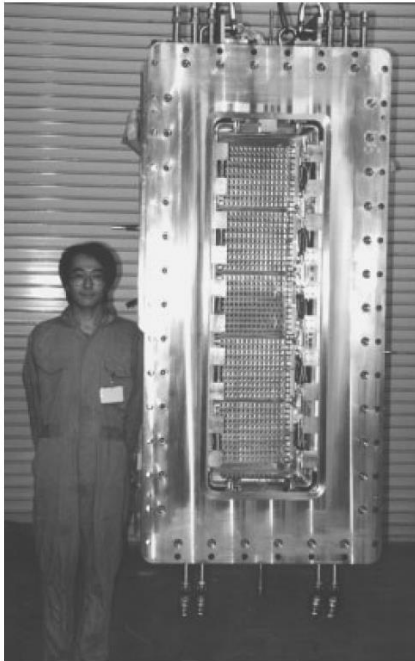


- Multi-stage acceleration
- Single-stage acceleration



The ITER neutral beam system: status of the project and review of the main technological issues, presented by V. Antoni

NBI system of the LHD fusion machine

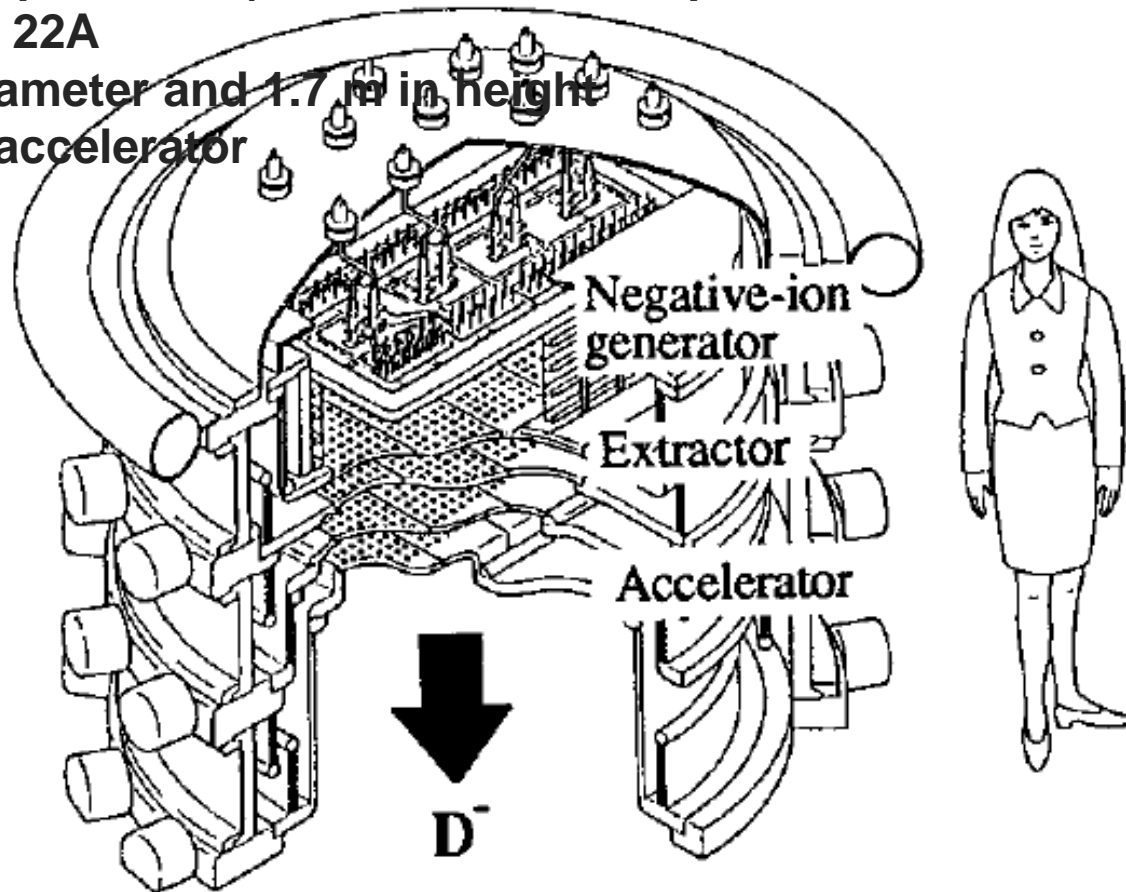


- 180 keV and 30 A
- Arc chamber: 35 cm x 145 cm, 21cm in depth
- Single stage accelerator

JT60U NBI system



- JT-60 (Japan-Torus) is a tokamak in Japan.
- 550 keV, 22A
- 2m in diameter and 1.7 m in height
- 3-stage accelerator

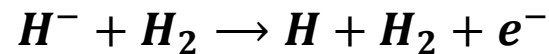


Neutralization



- **Gas neutralization**

- **Collisions between fast negative ions and atoms**

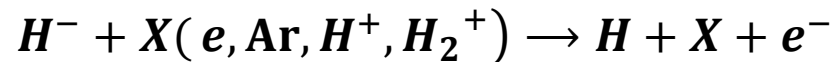


- **Fast ions can lose another electron after neutralized**



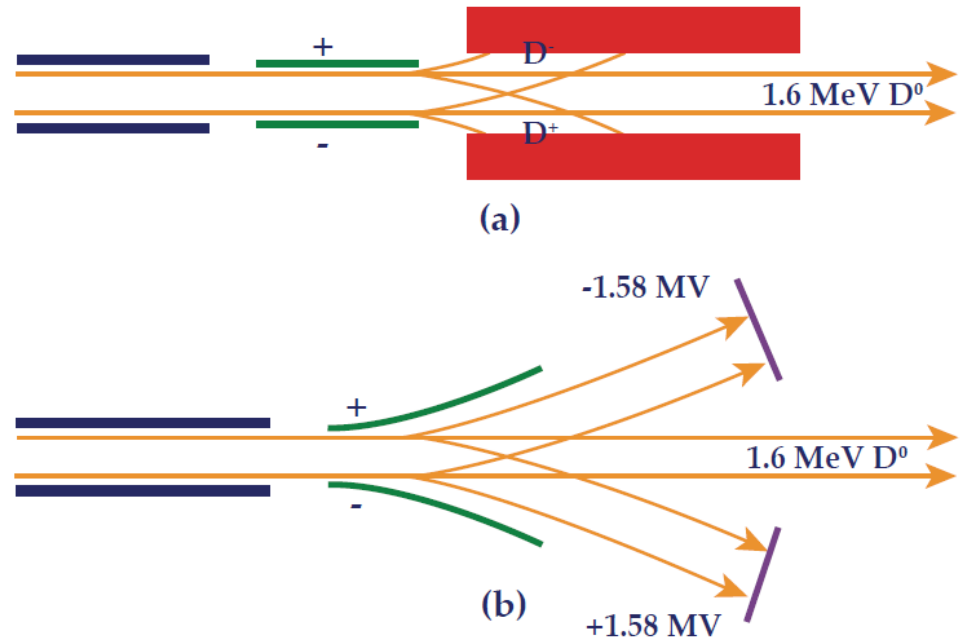
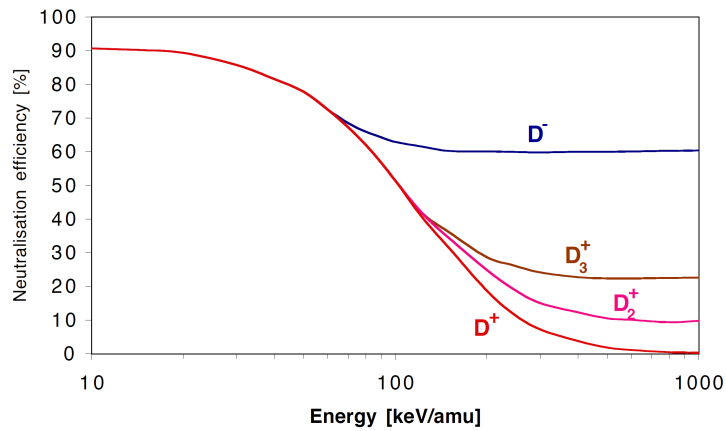
- **Plasma neutralization**

- **Collisions with charged particles in plasma**



- **The efficiencies reach up to 85% for fully ionized hydrogen plasma**

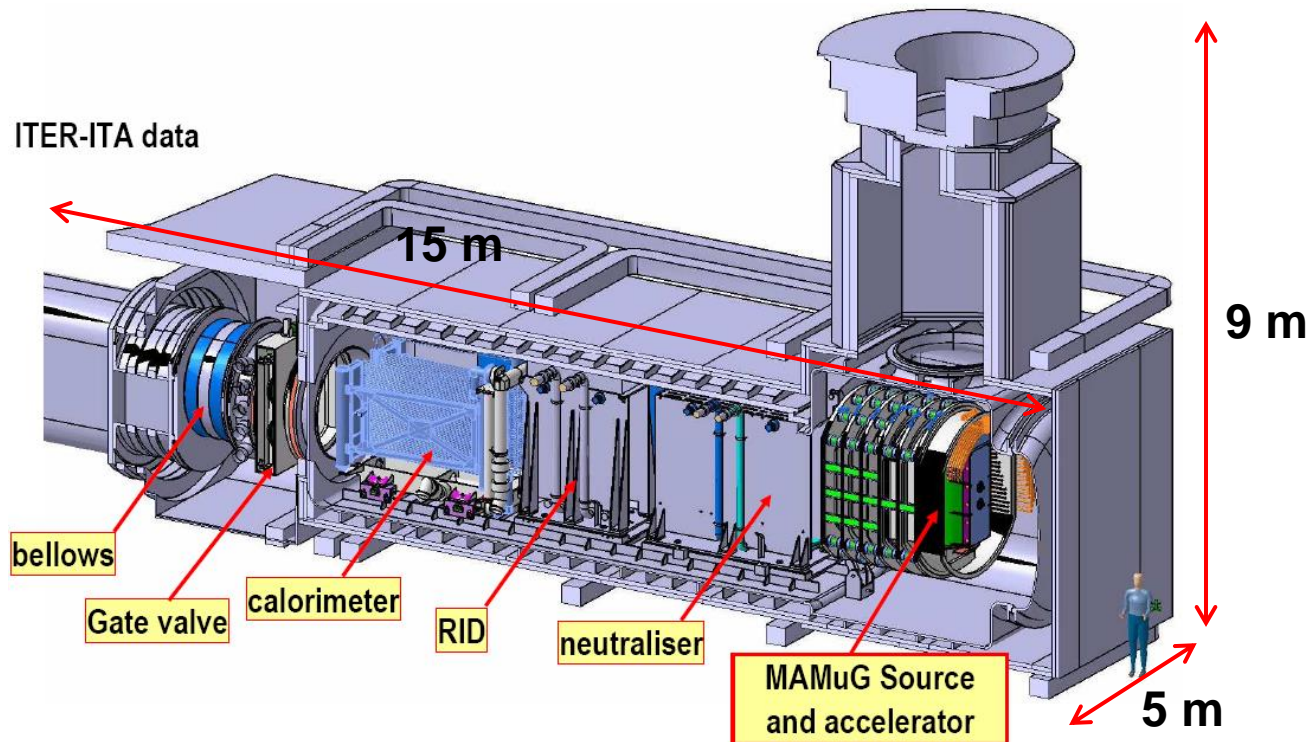
Beam dump



NBI for ITER

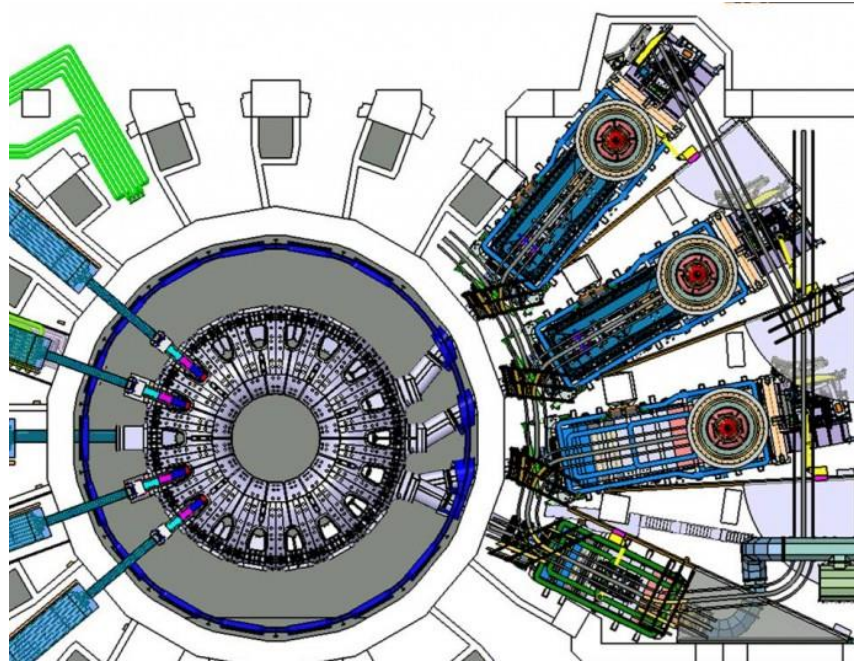


- beam components (Ion Source, Accelerator, Neutralizer, Residual Ion Dump and Calorimeter)
- other components (cryo-pump, vessels, fast shutter, duct, magnetic shielding, and residual magnetic field compensating coils)



The ITER neutral beam system: status of the project and review of the main technological issues, presented by V. Antoni

Neutral beam penetration



- **Parallel direction**
 - Longest path through the densest part of the plasma
 - Harder to be built
- **Perpendicular direction**
 - Path is short
 - Larger perpendicular energies leads to larger losses
 - Easier to be built